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WEAR RESISTANT COATINGS FOR REACTOR . / **COMPONENTS IN LIQUID SODIUM ENVIRONMENTS**

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ABSTRACT

Rubbing surfaces of critical components for the coming **generation of the nation's 1iquid-metal-cooled-fast-breeder reactors must withstand severe environments of high tempera**ture, liquid-metal corrosion, and nuclear irradiation. For some component requirements, the use of the better known **bearing materials and hardfacings has been shown to be in** adequate or impractical. For several applications, bulk (> 25 micron thick) coatings have provided the only successful solution to friction problems in high temperature sodium. **Coating processes that have been or are being evaluated in**clude plasma spray, detonation gun, spark-discharge, sputtering, and diffusion coating processes. Testing included friction and wear tests in up to 650°C sodium, sodium corrosion tests for up to one year, thermal cycling, bond strength tests, and irradiation tests in a fast reactor. For one

particularly severe application, a specially modified / **commercial coating of chromium carbide in a 15 volume percent nichrome binder, applied by a commercial detonation gun process, has so far been found to be most acceptable in meeting the** friction, corrosion, and irradiation damage criteria. Much of the test data reported is on this coating and its **antecedent modifications.**

V^ear Resistant Coatings For Reactor Componen-^4 In Liquid Sodium Environments

By R. N. Johnson, S. L. Schrock & G, A. Whitlow

INTRODUCTION

Rubbing surfaces of critical components for the coming **generation of the nation's liquid-metal-cooled-fast-breeder reactors must withstand severe environments of high tempera**ture, liquid metal corrosion, and nuclear irradiation. During the design of the Fast Flux Test Facility (the USAEC's sodiumcooled fast test reactor now being built at Hanford) it became **apparent that some rubbing surfaces in the core of the reactor** required materials capable of providing low coefficient of **friction in sodium at temperatures from 230°C to 650°C. Our** objective was to identify a material combination which would exhibit adequate service life in the reactor core environment and provide a friction coefficient of 0.4 or less at 230°C. This was found to be a stringent requirement since sodium is an extremely poor lubricant and is also exceptionally effective **in stripping oxide 1ayers from most material surfaces. The net** result is that many materials in contact with each other in sodium have the same high friction and self-welding tendencies **found in ultra-high vacuum environments. In some ways, vacuum** environments are considerably less severe, because sodium, in addition to enhancing materials self-welding, can be a highly

corrosive media in which mass transfer takes place and which precludes the use of nearly all known liquid or solid lubri**cants.**

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In order to identify materials capable of meeting the friction criteria, friction screening tests were performed on over **65 materials combinations, including metals, ceramics, and cermets both in sol id forms and as coatings applied by a variety of processes. During the early materials screening period, literature searches**⁽¹⁾ and review of earlier program *(2)* **results^** *'* **indicated that one of the most promising classes of materials that should be examined was the bonded carbides. '** *k*offman, et al $^{(2)}$ stated that after extensive screening of a wide variety of materials in wear tests in sodium, the bonded **carbides appeared to be the only materials as a class which con**sistently exhibited good wear and friction behavior at high **(above 500°C) temperatures in sodium.**

Subsequent friction screening tests confirmed that the bonded carbides provided the lowest frictions and in particular the chromium carbide base materials were selected for primary development efforts and titanium carbide base materials were selected for back-up development. A number of methods of attaching wear materials to the component surfaces were investi**gated and included mechanical attachments, v^elding, explosive bonding, and a variety of coating processes. Some method of coating appeared to o ffer the most desirable method of attachment** provided a reliable process could be found which met the qualification criteria. The qualification criteria were as follows:

- **The material shall have a coefficient of friction of** 0.4 or less in sodium at 230°C and no more than 0.9 **at 625°C.**
- **The coating shall not corrode more than 1/2 the coating** .thickness over its service life.
- **The material shall sustain the equivalent of 60 reactor scram cycles (cool from 625°C to 425°C in 1 minute in** sodium) without cracking, spalling or flaking.
- The material shall be capable of withstanding fast <code>neutron</code> exposures of 8 \times 10^{22} <code>n/cm² <code>without</code> <code>spalling</code></code> or flaking or failing to perform its function as a low **friction surface.**
- **The material shall not transmute to undesirable radio-คิก a ctive products (e .g . Co) which can contaminate downstream components and piping.**
- **The material shall be capable of being attached to core components in such a way that dimensional tolerances** are maintained and cold-work levels of substrate **materials are unchanged.** (20% cold-work is required in the 316 stainless steel fuel ducts for irradiation **swelling resistance).**

This paper describes the coating processes investigated

and the results of wear and friction tests, sodium corrosion tests, thermal cycling, mechanical integrity tests, and neutron irradiation tests performed in order to qualify candidate **materials and attachment process.**

DESCRIPTION OF COATING PROCESSES AND COATING COMPOSITIONS

Spark Transfer Coating

Spark transfer coating has its widest industrial usage **in the application of tungsten carbide films to complex geometry surfaces. Essentially the process consists of discharging stored energy from high voltage capacitors through an electrode** of the material to be deposited. In the resulting spark, a small amount of material is removed from the electrode and **welded to the substrate material. The process was found to be** capable of depositing coatings of a wide variety of materials in thicknesses of 2.5 microns (.0001") to 62 microns (.0025"). Little surface heating occurred (small parts could be hand held **during coating) so thermally induced distortion of parts or modification of bulk metallurgical structures was not a problem.** Bond strengths of carbide coatings applied by spark transfer appeared to be excellent in that qualitative tests such as severe bending and "hammer and chisel" tests were usually unsuccessful in dislodging any material. Capital equipment costs were low (state-of-the-art equipment was purchased for under \$5000). Deposition rate for full coating thickness was *2* **usually in the range of 1 to 2 in /minute.** As-deposited surface roughness ranged from about 60 p-in AA for well-optimized **tungsten carbide coatings to > 250** μ **-in AA for some of the**

1 ess-developed experimental chromium, titanium, and niobium carbide compositions and other cermets which were deposited **by the spark transfer process. Even with decidedly nonoptimum** coating parameters and surface finishes, the chromium carbide coatings showed friction coefficients in sodium which **averaged less than half those of the other carbides. These initial** results encouraged further development of chromium carbide base coatings by other deposition processes.

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Development of the spark transfer process for reactor applications has not progressed beyond the initial screening **studies due to the early success of concurrent developments (the b-gun coating described below) and time and manpower which would be necessary to develop the process to the same degree of optimization for chromium caribde coatings as had been done** for tungsten carbide. The process is still being considered **as a "back-up" process for further development as needs and resources allow.**

Sputtering

Coatings of chromium carbide + 15% nichrome were applied by sputtering to Type 316 stainless steel substrates. Coatings **were 4 to 5 mils (100 to 125p) thick and were deposited at 125°C and at 600°C at deposition rates of 23u/hr. The coatings appeared** to be very hard and tightly adherent in the as-deposited condition, 'but **after** heating **to 625°C to** simulate **in — reactor**

temperatures, both the low temperature and high temperature deposition coatings showed cracking with the low temperature coating showing both cracking and spalling. It is possible that further development effort, perhaps using graded coat**ings, could overcome the cracking problems; however, no further** work is anticipated at this time.

Plasma Spray

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A number of chromium carbide and titaniurn carbide base coatings on 316 stainless steel and Inconel 718 substrates were prepared by a non-transferred, D. C.-constricted plasmaarc with fully shielded effluent, using argon gas. The coatings exhibited good wear behavior, but, as shown below, were eliminated for in-reactor applications by the cracking and spalling tendencies found under both irradiation and thermal cycling tests.

Detonation Gun

The detonation-gun coating of chromium carbide plus 15 vol. *%* **nichrome has been s e le c te d as the reference process and material** for the coating of reactor core components for the FFTF. In the detonation gun process, the powdered coating material is in**troduced into the chamber of the gun along with a metered** quantity of oxygen and acetylene. The mixture is detonated by a spark plug and in the resulting explosion, the powdered materials are heated to a plastic or molten state and accelerated down **the gun barrel to impinge on the substrate at approximately three**

times sonic velocity. The combination of hot particles and the high kinetic energy of impact creates a coating which is typically more dense and which has a higher bond strength than similar coatings applied by plasma spraying. Much of the test data presented in this paper will be on coatings applied by **the detonation gun technique.**

Materials Selected for Evaluation

After friction screening tests had been completed, materials remaining for final testing and evaluation included chromium carbide and titanium carbide coatings, each containing 15 vol.% of a binder. Typically, coatings were 75 to 125 µ **(3 to 5 mils)** thick and were applied to grit-blasted 316 SS **or Inconel 718 substrates.** The materials and compositions are **listed in Table 1. Although chromium carbide coatings showed** the lowest friction, titanium carbide was also included in case sodium corrosion tests showed excessive attack on the chromium carbide materials. As will be shown later, an unexpected result was that the titanium carbide compositions showed the more severe sodium exposure effects.

Carbide Cyating Compositions Evaluated

*** Inconel 718 - TM - International Nickel Co.**

TESTING PROCEDURES **AND RESULTS**

Friction and Wear Tests

Wear tests were conducted by Atomics International, **Liquid Metal Engineering Center and Westinghouse Advanced** Reactors Division (3, 4, 5) in a cooperative program with the Hanford Engineering Development Laboratory. In all cases, **t e s t s were conducted over temperature ranges from 200° C to 625°C in high purity sodium (oxygen content less than 5 ppm** measured by total amalgamation or less than 1 ppm measured by vanadium wire analysis). Test specimen geometry varied somewhat at each of the three test sites, but all used a general pin en-plate, oscillatory rubbing with flat-on-flat interface **configurations. Loads ranged from 300 psi to 1000 psi for most tests.**

Figure 1 shows some of the early data developed in screening tests which led to the further testing and development of **chromium carbide coatings. The chromium carbide coatings** *vere* the only coatings or materials found which consistently met **the friction criteria over all temperature ranges.**

Surface finish was found to have an effect on the friction behavior of the chromium carbide coatings. The best friction results were obtained on coatings which were brush finished with a fine silicon carbide impregnated nylon brush to a finish of 80 to 125 u-in AA. The resulting surface texture is best described as a smooth, "pebbly" finish. When the coating was applied by the spark transfer technique, a surface finish of 80 to 250 u-in AA or rougher was obtained which resulted in inconsistent friction coefficients averaging higher than the **brush-finished D-Gun coating or a lapped-finished D-Gun coat**ing. A lapped finish of 5 to 16 u-in AA also resulted in in² consistent friction behavior intermediate between the brush**fin ish ed and the spark-transfer coatings.**

Qualification tests of over 1000 inches of rubbing in sodium at 625°C and loads up to 1000 psi have shown no signif**icant wear on the chromium carbide coatings.**

Sodium Compatibility Tests

Sodium compatibility testing is necessary to assure (1) that the chromium carbides are chemically stable and do not dissociate over a period of time in sodium, and (2) that the leaching of binder materials, such as nickel and chromium from nichrome, does not result in a physical breakdown of the coating during the service life of the component.

As a result of these potential problems, a portion of the present program is directed at evaluating the effects of long **term (8000 hours) high temperature (625°C) sodium exposure on** the various coating combinations. This program ⁽⁶⁾ is being conducted in austenitic stainless steel systems which include

provisions for impurity monitoring and control. Test conditions encompass a hot[/]leg test temperature of 625°C, with a velocity of **1 f p s , a loop temperature differen ce (aT) of 238°C, and a sodium oxide level of 0.5 to 1.0 ppm as determined by VWED (Vanadium Wire Equilibration Device).**

Samples (0.75" x 0.75" x 0.180" with a 0.003" to 0.005" thick coating on both squared faces) are periodically withdrawn **from the system hot leg into an argon atmosphere. Residual** sodium is removed from the samples by successive rinses in **alcohol and water. After vacuum drying and microv^eighing, sodium** effects are determined by chemical, metallographics and electron **microprobe analyses. Average corrosion rates in mils of thickness l o s t per year (mpy) are calcu lated on the basis of weight loss and coating density values.**

Titanium carbide base coatings exhibited high corrosion rates due to coating instability, and were soon eliminated **from the program. The as-deposited titanium carbide showed a non-stoichiometric composition of TiCg ^ before sodium exposure** and a stoichiometric TiC composition after exposure. The coating failune was attributed to the stresses created by the volume **changes occurring during carburization of the coating. (Sodiurn maintains an equilibrium carbon level by removal of carbon from the stainless steel loop system.)**

Early chromium carbide coatings suffered from coating process control problems resulting in some specimens exhibiting **cracking through a contaminated layer in the coating a fter**

2000 hrs sodium exposure at 625°C. Nevertheless, some speci**mens attained 8000 hours exposure time** and **exhibited a corrosion rate of 0,3 mpy. After the** process **control problems were resolved, exposure to sodium of the** later **chromium carbide combinations resulted in time** dependent weight **losses with average corrosion rates of** approximately **0.25 mpy.** Figure 2 shows a comparison of the chromium carbide and titanium **carbide corrosion rates.**

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Although extensive sodium penetration of the coatings occurs, no changes in cross sectional microstructure of the D-qun **applied chromium carbide coatings were observed. The weight losses were attributed to loss of binder** material **to the sodium** possibly coupled with loss of chromium carbide. Surface analyses **showed, for example, that the reference** chromium **carbide/ nichrome (LC-IH) coating lost nickel from the nichrome binder** to the sodium, but this did not result in marked surface deterioration. Some internal cracking was observed, however, **in the plasma sprayed CM and CN-IP coatings at exposure times greater than 1000 hours.**

Thermal Cycling Tests

An inherent problem with many coatings is a tendency to physically disintegrate during thermal transients. Hence, a third part of the present program involved tests designed to simulate temperature fluctuations experienced by the reactor **component during service.** The experimental facility used in

this work is similar to the sodium compatibility loops, except that the loop is equiped such that by mechanical manipulation, test specimens can be transferred between the 625°C **hot leg and an adjacent 427°C sodium quench tank, without** coming into contact with air. A thermal cycle consists of **holding a specimen at 625°C for a minimum of 8 hours and a maximum of 4 days, quench to 427°C in approximately 1 minute,** raise to 625°C at a rate of 28°C/hr. Specimens exposed to this in-sodium cycling are removed after 15, 30 and 60 cycles directly into an argon atmosphere and cleaned as described in the previous section. Temperature cycling effects are then **determined metallographically.**

D-gun applied chromium carbide coatings have shown no deterioration as a result of thermal cycling. Plasma sprayed **chromium carbide/molybdenum, CM coating, has, however, shown** coating disintegration after 29 thermal cycles. The fracture **occurred** near the coating/substrate interface and is attributed to the mismatch in thermal expansion coefficients between the CM coating and the 316 stainless steel substrate.

Irradiation Tests

Small discs of 316 stainless steel and Inconel 718 were **coated on one side with a candidate chromium carbide material** and were inserted in stainless steel capsules filled with sodium. The capsules were sealed by welding and were irradiated in the EBR-II reactor in Idaho Falls. A number of **capsules are now being irradiated to fluences as high as** *² ? ?* **³X 10 n/cm , Two capsules have been removed for examination so far, after receiving neutron fluences of approximately** *97 9* **¹X 10 n/cm . Irradiation temperatures were 450°C (844°F)** and 585°C (1083°F) respectively. The materials in Table 1 **which have been irradiated and examined so far include LC-IH, CI-+P, CI and CM coatings.** In addition, the LC-1C and CI-D **coatings are being irradiated but are not yet available for examination. The titanium carbide coatings {TM and TN) were eliminated from consideration by corrosion test results and** thus were not included in the irradiation tests.

Post-irradiation examination and testing⁽⁷⁾ included visual **apd low-magnification examination, weight and thickness change measurements, optical metallography, scanning microprobe exam**ination, x-ray analysis, and bond strength testing.

Visual examination revealed that the plasma sprayed coatings of all compositions showed tendencies for cracking and spalling, both within the coating and at or near the coating-

substrate interface. The observed effects ranged from barely detectable cracks or no externally observable cracks on some of the CN-1P (chromium carbide - 15% nichrome) coated speci**mens to essentially complete removal of the coating on the CM (Chromium carbide - 15 Vol. % Mo) coated specimens. All the** plasma-sprayed coatings showed at least some cracking within the coating and at the interface under metallographic examination, however. Figure 3 is an example of the cracking found **in the CN-IP specimens.**

None of the 24 D-gun coated specimens showed any cracking or spalling tendencies by either visual or metallographic examination. Figure 4 shows the D-gun applied chromium carbide - 15% nichrome coating (LC-1H) after irradiation. The voids seen in the coating interface resulted from the "pull-out" of Al₂O₃ **p a r tic le s during metallographic preparation. The AI2O2 particles** were eliminated in later coatings by improvements in the precoating grit blasting operation. There were no observable **differences between the irradiated and un-irradiated conditions of the D-gun coatings when examined by metallography or by electron microprobe.** X-ray results showed no compositional changes and only slight line sharpening of the pattern after **irradiation.** Bond strength tests were conducted by an epoxy lift-off tensile test. In all cases, both before and after **irradiation, failures occurred in the epoxy at tensile strengths** up to 13,000 psi. Perhaps one of the more significant indications of the coating integrity was found unintentionally when **attempts to remove samples of coating by hammer and chisel methods for X-ray analysis proved unsuccessful.**

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SUMMARY AND CONCLUSIONS

Wear tests have shown chromium carbide coatings to have the lowest friction and best wear resistance in sodium at tempera**tures from 200°C to 625°C of any of a wide variety of materials te sted . The next best material from a wear performance viewpoint, titanium carbide-base coatings, fa iled the sodium** corrosion tests due at least in part to carbon loss during **deposition, and subsequent carburization during sodium exposure.** Further friction tests, sodium corrosion tests, thermal cycling tests and irradiation tests resulted in the selection of a chromium carbide - 15 Vol.% nichrome coating applied by detonation gun as the only material and coating process which has so far passed all qualification tests for usage on stainless steel components in liquid-metal-cooled nuclear reactors. **An identical composition coating applied by a plasma spray pro-22 cess failed to remain intact during irradiation to 1 x** 10²² $\frac{1}{2}$ $\frac{1}{2}$ $\frac{2}{3}$

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FIGURE CAPTIONS

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Figure 1. Friction ranges observed for various materials rubbing in sodium.

Figure 2. Corrosion of chromium carbide and titanium carbide coatings in 625°C sodium.

Figure 3. Cracking found in plasma-sprayed chromium carbide 15 Vol.% nichrome coating on 316 SS after irradiation to 1 X 10^{22} n/cm².

Figure 4. Detonation-gun coating of chromium carbide - 15 Vol.² **22 2** *nichrome on 316 SS after irradiation to 1 X 10²² n/cm².*

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