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XPS, XRD and SEM Study of Oxidation of UO₂ by Air in Gamma Radiation at 150°C

Analyses diffractométriques, par spectroscopie XPS, et par MEB de l'oxydation de l'UO₂ par l'air dans un champ de rayonnement gamma à 150°C

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XPS, XRD AND SEM *STUDY* OF OXIDATION OF UO² BY AIR IN GAMMA RADIATION AT 150°C

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ABSTRACT

UO₂ disks were heated at 150°C in air, in O₂ with 60% saturated steam, and in Ar with 60% saturated steam atmospheres for \sim 2 years in gamma fields equivalent to those associated with 10- to 20-year-old used CANDU fuel. Surface analysis of the disks, using X-ray photoelectron spectroscopy, X-ray diffraction and scanning electron microscopy, shows formation of U_3O_8 on the UO, disks exposed to air or O,. This is the first report of the formation of U_3O_8 on UO, by air oxidation at such a low temperature. The rate of U_3O_8 formation by dry air oxidation of $UO₂$ at 150°C and in gamma fields of dose rate ~15 Gy \cdot h⁻¹ is very low. The presence of water vapour along with O_2 increases the oxidation of UO_2 in gamma fields, leading to the formation of $\rm U^{6+}$ phases, e.g., $\rm UO_3$ •x $\rm H_2O$ along with $\rm U_3O_8$. On the other hand, $UO₂$ disks did not suffer any oxidation by water vapour radiolysis at 150°C in an O,-free (60% saturated steam in Ar) atmosphere. These are important observations for the dry storage of used fuel because the oxidation of UO, to U_3O_8 is accompanied by a volume expansion, which could result in splitting of the Zircaloy cladding and powdering of the fuel matrix.

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ANALYSES DTFFRACTOMETRIQUES. PAR SPECTROSCOPIE XPS. ET PAR MEB DE L'OXYDATION DE L'UO, PAR L'AIR DANS UN CHAMP DE RAYONNEMENT GAMMA À 150 °C

par

S. Sunder et N.H. Miller

RESUME

Des disques d'UO₂ ont été chauffés à 150 °C dans l'air et dans l'UO₂ en présence de vapeur saturée à 60 %, et dans l'Ar en atmosphères de vapeur saturée à 60 % pendant environ 2 années dans des champs de rayonnement gamma équivalents à ceux produits par du combustible CANDU irradié et stocké depuis 10 à 20 ans. L'analyse de la surface des disques, par les méthodes de spectroscopie de photoélectrons XPS, de diffraction des rayons X et de microscopie électronique à balayage a révélé la formation d' U_3O_8 sur les disques d'UO₂ mis en contact avec l'air ou l'O₂. Il s'agit de la première fois que l'on signale la formation d'U₃O₈ sur l'UO₂ par oxydation par l'air à une température aussi basse. La vitesse de formation d'U₃O₈ résultant de l'oxydation de l'UO₂ par l'air sec à 150 °C et dans des champs de rayonnement gamma à un débit d'environ 15 Gy·h⁻¹ est très faible. La présence de vapeurs d'eau et d'O₂ accroît l'oxydation de l'UO₂ dans les champs de rayonnement gamma et conduit à la formation de phases U^{ω} , p. ex., UO_3 •x H_2O et U_3O_8 . Par ailleurs, les disques d'UO₂ n'ont subi aucune oxydation par radiolyse de la vapeur d'eau à 150 °C en atmosphère exempte d' $O₂$ (vapeur saturée à 60 % dans l'Ar). Les faits observés revêtent une grande importance pour le stockage à sec du combustible irradié, car l'oxydation de l'UO, en U_3O_8 s'accompagne d'une dilatation qui pourrait conduire à l'éclatement de la gaine en Zircaloy et à la pulvérisation de la matrice de combustible.

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AECL-11351 COG-95-296

CONTENTS

 $\sim 10^{11}$ km $^{-1}$

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 $\sim 10^{11}$

LIST OF TABLES

Page

 $\hat{\mathcal{E}}$

LIST OF FIGURES

 $\mathcal{L}_{\mathcal{A}}$

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LIST OF FIGURES (concluded^

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1. INTRODUCTION

Dry storage of used nuclear fuel in above-ground containers offers an economical option for short- to medium-term storage until a permanent disposal facility becomes available. Used fuel is already being stored in concrete canisters at several Canadian reactor sites [1-5 and references therein].

The performance of used CANDU¹ fuel in a dry-storage facility will be affected by the extent of air oxidation of any defected fuel bundles. Oxidation of UO_2 fuel to U_3O_8 (or UO_3) results in a volume expansion [6] that could cause powdering of the fuel and splitting of the Zircaloy cladding. Therefore, the oxidation of used fuel bundles with deliberately introduced defects, at somewhat higher than anticipated storage temperatures, is being investigated at Whiteshell Laboratories [3-5]. These are called "Controlled Environment Experiments" (CEX). The CEX-1 experiment is investigating the storage of used fuel in dry air, while the CEX-2 experiment is investigating storage in moisture-saturated air. Both are being conducted at 150°C. The results from CEX-1 were used to develop a model for predicting the progress of oxidation of defected fuel elements in a dry-storage facility [7]. This model predicted that the oxygen in the storage container would be consumed very rapidly, at a rate about two orders of magnitude larger than the rate calculated using literature data on UO₂ oxidation. To resolve this discrepancy, it is necessary to have accurate data on fuel oxidation.

The radioactivity associated with used fuel samples [8] limits the handling and analysis of the samples to determine the extent of oxidation. One must use small samples of used fuel to reduce the exposure of operator and equipment to radiation [5,9]. Therefore, we have studied the oxidation of unirradiated CANDU fuel exposed to selected controlled atmospheres at ~150°C in gamma fields equivalent to those associated with used CANDU fuel. These studies complement the air oxidation studies of irradiated CANDU fuel [3-5] and of unirradiated fuel without any applied gamma fields [1,6,10-14], being carried out at Whiteshell Laboratories. The results of these investigations are presented here.

$2.$ EXPERIMENTAL

2.1 MATERIALS

 $UO₂$ samples were obtained from an unused CANDU fuel bundle. The fuel pellets consist of sintered polycrystalline UO₂, with a density of ~10.6 g•cm⁻³, i.e., > 96% of theoretical density. The pellets (diameter 13 mm) were cut into -3 mm-thick disks using a diamond saw. The estimated weight of the disks was ~4 g each.

The disks were mechanically polished on all sides, including the rim, using 600-grit SiC paper, to expose a fresh UO_2 surface [15]. An identifying letter was engraved on the bottom face of each disk to distinguish between samples. The disks were stored in an inert atmosphere glovebox prior to being sealed in specially designed glass vials to minimize oxidation by atmospheric $O₂$.

^{&#}x27;CANada Deuterium Uranium, registered trademark of AECL

Four types of experiments were carried out:

There was no water deliberately added to the vials used in Experiments A and B. The air sealed in the vials in Type B contained ambient humidity. The vials were constructed from -20 mm o.d. Pyrex glass tubes, wall thickness 1.5 mm. The wide bottom ends of the vials were sealed after the polished disks were placed in the tubes. The narrow necks of the vials were left unsealed for Experiment A (Figure la) and sealed for Experiments B, C and D after the vials were purged with the desired gas (Figures Ib and le). For Experiments C and D, water was added to each vial, using a calibrated Eppendorf pipette attached to a thin Teflon tube, after purging with $O₂$ (Experiment C) or Ar (Experiment D) but before sealing the top end. The inner lip near the bottom of the vials used in Experiments C and D (Figure le) ensured that the UO₂ disks were not in direct contact with liquid water prior to heating.

The glass vials, containing polished UO., disks, were placed in a specially designed aluminum vessel, Figure 2. The vessel was placed in the central position between four vessels containing used CANDU fuel bundles, in a shielded canister in the Immobilized Fuel Test Facility (IFTF) at the Whiteshell Laboratories [5,17], Figure 3. Thus, UO_2 disks in the glass vials experienced essentially the same gamma field as did the used CANDU fuel bundles. The gamma field at the centre of the vials is estimated to be about 15 Gy/h, similar to that expected for 10- to 20-year old CANDU fuel (Appendix A). The temperature of the canister was raised to 150°C over a period of about 2 days. Power to the canister heater was turned off every three weeks to sample the atmosphere in the used fuel vessels. Downtime on each occasion was ~3 days. The UO₂ samples were exposed to this heating (3 weeks at 150°C) and cooling (3 days at $\sim 30^{\circ}$ C) cycle for 2.1 a, then retrieved.

The aluminum vessel containing the sample vials was removed from the IFTF canister after it had cooled down to room temperature and its outer surface was decontaminated. The glass vials, containing U02 disks, were checked for contamination and transferred into a nominally $O₂$ -free, inert-atmosphere box. The sample disks were taken out of the glass vials by breaking the vials in the inert atmosphere box. They were then mounted on the XPS specimen holder and brought into the X-ray photoelectron spectrometer for analysis [15,18], The surfaces of

FIGURE 1: Schematic Diagrams of Glass Vials Used to Study Oxidation of UO₂ in Gamma Fields at High Temperatures: (a) Open to Atmosphere (Experiment A); (b) Sealed with Air (Experiment B); (c) Sealed with O_2 or Ar and Water (Experiments C and D)

FIGURE 2: Schematic Diagram of Experimental Set-up (Vessel with UO₂ Vials)

FIGURE 3: Schematic Diagram of the Relative Placement of the Vessels Containing $UO₂$ Samples and Used CANDU Fuel: A, Used Fuel Container; B, UO₂ Container; and C, IFTF Canister

the disks were analyzed using X-ray photoelectron spectroscopy (XPS), X-ray diffraction (XRD) and scanning electron microscopy (SEM), in that order.

2.2 X-RAY PHOTOELECTRON SPECTROSCOPY

The XPS studies were carried out using a PHI-5300 ESCA system, which allows one to transfer samples from the inert atmosphere chamber to the evacuated sample compartment of the XPS spectrometer without exposure to air using a vacuum transfer vessel [18]. The XPS spectra were excited using Mg K α radiation filtered through an Al window. The energy scale of the spectrometer was calibrated using the following line positions of noble metals: Au $4f_{\gamma/2}$, 84.0 eV; Ag $3d_{5/2}$, 368.3 eV; and Cu 2p_{3/2}, 932.56 eV [19-21].

2.3 X-RAY DIFFRACTION

After XPS analysis, XRD data were obtained from the complete sample disks, using a Rigaku Rotaflex X-ray diffractometer equipped with a 12-kW rotating-anode Cu K α X-ray source and a diffracted-beam graphite monochromator. The diffractometer was calibrated using a silicon standard from the U.S. National Bureau of Standards (NBS)². The diffractometer scanning rate was $1^{\circ}(2\theta)$ -min⁻¹. The data were acquired using a Rigaku IBM PC (Version 3.0, June 1992) data acquisition system and analyzed using JADE+ software obtained from Material Data Inc. of Livermore, CA, U.S.A.

2.4 SCANNING ELECTRON MICROSCOPY

The sample surfaces were examined using a JEOL JSM-6300V scanning electron microscope after XRD analysis. The microscope was equipped with a $LaB₆$ electron gun. The micrographs were recorded using an accelerating voltage of 15 kV, beam diameter ~50 μ m, and sample current ~4 pA. The specimens were gold-coated to minimize surface charging.

3. RESULTS AND DISCUSSION

3.1 VISUAL OBSERVATIONS

All disks were recovered intact from the vials. In Experiment D $(Ar + water)$, water droplets were found on the surface of the disks as well as on the inner walls of the glass vials at the end of the Experiment; we thus conclude that there were no leaks in these vials. The disks used in Experiment C (O_2 + 60% ss) were generally darker in colour than the other disks. One of the sample disks in Experiment C (labelled X in Table 1) had a slight yellowish surface coloration.

 \overline{a}

The present name of the NBS is National Institute of Standards and Technology (NIST)

TABLE 1

<u>XPS RESULTS FOR THE SURFACE OXIDATION OF UO, DISKS</u> AT 150°C IN GAMMA FIELDS

a ss means saturated steam at 150°C.

This Experiment's atmosphere was probably equivalent to A (see text).

 $\pm \pi$ This sample's atmosphere probably contained a trace of O_2 impurity (see text).

3.2 X-RAY PHOTOELECTRON SPECTROSCOPY

Low-resolution survey spectra were recorded for the 0- to 1100-eV region to determine the elements present in the sample surface and to check for surface contamination. High-resolution spectra were recorded for the U 4f, valence bands (0-20 eV), O Is and C Is regions to determine the chemical state of these elements (the carbon signal was due to ubiquitous hydrocarbons present on solid surfaces). The C Is band was used to correct for charging with a binding energy value of 284.8 eV for this peak [15].

The band due to the U 4 $f_{7/2}$ core level is very sensitive to the chemical state of the uranium atoms, and can be quantitatively resolved into U^{6+} and U^{4+} components to determine their relative amounts in the XPS sample, as described in references 20, 22 and the references therein. The relative area of the U^{6+} and U^{4+} peaks is a direct measure of the oxidation state of the uranium atoms in the sample surface. Results obtained from the analysis of the U *4flp* band were verified by analyzing the relative intensities of the bands in the valence region [20,22-24]. A U⁴⁺ atom has two electrons in its 5f valence shell. The band seen around 1 eV in the XPS spectra of uranium compounds is assigned to these electrons [15,20,23-25]. Oxidation of U⁴ compounds, e.g., UO₂, results in the loss of these electrons from the uranium atom and is indicated in the spectrum by a decrease in the intensity of this band. Intensity measurements are not affected by any error in the binding energy scale, and thus provide an independent check on the results from the analysis of the U $4f_{\gamma}$, band.

Figures 4 and 5 show the spectra for the U 4f and valence regions, respectively, for one sample each from Experiment C (sealed vials, $O_2 + 60\%$ ss, sample X) and Experiment D (sealed vials, Ar + 60% ss, sample V). The spectra shown in Figures 4a and 5a are typical of those observed for U⁶ oxides, i.e., UO₃, [24,26] while the spectra shown in Figures 4b and 5b are typical of those seen for U^{4*} oxides, i.e., UO , $[23,24,27]$. Thus, the U 5f band has high intensity in the spectrum of sample V, Figure 5b, while it shows very little intensity in the spectrum of sample X, Figure 5a. Deconvolution of the U 4 $f_{\gamma/2}$ band into U^{6+} and U^{4+} components indicates that the band for sample X (Experiment C) can be accounted for mainly by a U⁶⁺ feature (U⁶⁺/U⁴⁺ ratio ~100), while that for sample V, (Experiment D) can be accounted for primarily by a U⁴⁺ feature (U⁶⁺/U⁴⁺ ratio ~0.01), Table 1. The U 4f₇₁₂ bands in the XPS spectra of one sample from Experiment A (sample M) and one sample from Experiment B (sample L) are analyzed in Figure 6. These spectra yielded U^{6+}/U^{4+} ratios of -2.8 and 6.2, respectively.

Table 1 gives all the XPS results, expressed as uranium oxidation states, obtained by analyzing the U $4f_{\eta_2}$ band. In all samples exposed to air or O_2 (Experiments A, B and C) the $U^{6'}/U^{4'}$ ratio is ≥ 2 , indicating that the surface of these samples has been oxidized to phase(s) with average oxidation state(s) equal to or higher than that in U_3O_8 (the U^{6+}/U^{4+} ratio is equal to 2 in U_3O_8). The highest ratios are seen for Experiment C (O_2 + 60% ss), Table 1. The surface of sample X , as seen by XPS, contains uranium almost entirely in the $+6$ state. In contrast, the samples exposed to water vapour radiolysis at 150° C in O₂-free atmosphere (samples U and V, Table 1) appear to have undergone no. oxidation. To confirm this observation, we analyzed both faces of the sample disks used in Experiment D (samples U, V and O). Both faces of disks U and V show no oxidation. However, one of the samples in Experiment D, disk O, shows some oxidation, i.e., U^{6}/U^{4} ratios of 0.3 and 0.4 for its two faces. These values are less than the value of 0.5 expected for U_3O_7 . We believe that the slight oxidation suffered by this sample is probably due to the presence of traces of $O₂$ impurity, which could have entered the vial during the glass-sealing step.

FIGURE 4: XPS Spectra for the U 4f Region of $UO₂$ Disks Exposed to Gamma Fields at 150°C in: (a) O_2 + 60% Saturated Steam (Experiment C, Sample X); and (b) Ar+ 60% Saturated Steam (Experiment D, Sample V)

FIGURE 5: Valence Band Region in the XPS Spectra of UO₂ Disks Exposed to Gamma Fields at 150°C in: (a) O_2 + 60% Saturated Steam (Experiment C, Sample X); and (b) Ar + 60% Saturated Steam (Experiment D, Sample V)

BINDING ENERGY (eV)

FIGURE 6: Resolution of the U 4 $f_{7/2}$ Band into $U^{6'}$ and $U^{4'}$ Components for UO₂ Disks in Air: (a) Open Vial (Experiment A, Sample M); and (b) Vial Sealed with Air (Experiment B, Sample L)

The U⁶⁷/U⁴⁴ ratio for one of the samples in Experiment C, sample Z, is significantly lower than those observed for the other two samples in this Experiment (Table 1). The U^{6+}/U^{4+} ratio for this sample is similar to those observed for the samples used in Experiment A (open air). One possible explanation for this result is that the glass vial for this sample may have contained a leak, resulting in the loss of water on heating and rendering the Experiment equivalent to type A (open air) rather than type C $(O_2 + 60\% \text{ ss})$. The XRD results for this sample are in agreement with these XPS results (see below).

The U⁶⁺/U⁴⁺ ratios are higher in Experiment B (air-sealed vials) than Experiment A (open vials) even though the air (O_2) available to the samples in Experiment A was nominally unlimited. The XRD results are consistent with this observation (see below).

The O 1s spectra of all the samples indicate the presence of both metal-oxide $(O²)$ and hydroxyl and water species in the sample surfaces, as illustrated in Figure 7 for sample M (Experiment A). The procedure to resolve the O 1s band into $O²$, OH and H₂O components has been discussed elsewhere [20,28]. All the samples showed the presence of OH and H₂O features, in addition to the O^2 feature, in the O 1s band in the XPS spectra.

BINDING ENERGY (eV)

FIGURE 7: XPS Spectrum for the O 1s Region of a UO₂ Disk Exposed to Gamma Fields at 150°C in Air (Open Vial, Experiment A, Sample M)

3.3 X-RAY DIFFRACTION

X-ray patterns were recorded for the 2θ range between 10 and 120 $^{\circ}$. XRD patterns were not recorded for sample U, which (like sample V) experienced no oxidation according to the XPS results discussed above (Table 1).

The use of XRD to detect the products formed on UO_2 surfaces during oxidation has been described by several authors [e.g. references 10,12,13,29-32]. Because the unit cells of U_4O_9 and U_3O_7 are similar to that of UO_2 , the oxidation of UO_2 to U_4O_9/U_3O_7 results mainly in the distortion and/or displacement of the XRD features [10,29-34]. UO, has a cubic unit cell with $a = 5.4701$ Å, while U_4O_9 also has a cubic unit cell with a only slightly shorter than in UO₂ (5.44 Å); and the three U₃O₂ phases (α , β and γ) have tetragonal unit cells with c/a values very close to 1 and with *a* very close to that in UO, [30]. In contrast, oxidation to U_3O_8 or beyond (e.g., UO_3) causes the appearance of quite distinct features due to major changes in unit cell size and symmetry in going from UO, to U_3O_s (or UO_3) [6,11,12,27,29,30,35]. Thus, the formation of U_3O_8 on UO, samples is easily detected in XRD by the appearance of characteristic 001 and 130/200 features of U_1O_s , with *d* values of \sim 0.415 nm and 0.34 nm, respectively, (i.e., 20 values of \sim 21° and \sim 26°, respectively, for CuKa X-rays).

Figure 8 shows XRD patterns of representative samples from the four types of Experiments in the present study. The *d* spacings obtained from the XRD patterns shown in Figure 8 are listed in Appendix B. Figure 9 shows the region expected to contain the distinctive features of U_3O_8 (15 to 40°) on an expanded scale. The samples exposed to air or O_2 show the characteristic features attributed to U_3O_8 (see a, b and c in Figures 8 and 9). The 001 feature of U_3O_8 ($d \sim 0.415$ nm, $2\theta \sim 21^\circ$) is very intense in the XRD patterns of samples heated in air (Experiments A and B), Figures 8 and 9. This feature has the highest peak height in the XRD patterns of samples in Experiment B (closed vials containing air). The disproportionately high intensity of this feature is probably related to the preferential growth of U_3O_8 in the (001) direction on UO, grains with a (111) orientation [13,27,35]. The XRD patterns of the samples exposed to $O_1+60\%$ ss (Experiment C) are dominated by $UO_3 \cdot xH_2O$ phases, Figures 8c and 9c. In contrast, the samples exposed to Ar and $H₂O$, (Experiment D) including sample O, do not show these characteristic features of U_3O_8 in their XRD scans (Figures 9a and lOa).

Crystalline phases present in the surface of the samples were identified using the JADE+ computer software by phase matching with the powder diffraction patterns in the files of the Joint Committee for Powder Diffraction Standards [36]. Table 2 lists the major and minor uranium oxide phases identified by their XRD patterns. It should be noted that the major phases present in the surface of samples W and X, Experiment C $(O_2 + 60\%$ ss), are uranium trioxides, i.e., U oxidation has gone beyond the U_3O_8 stage, a result consistent with the XPS results for the two samples (Table 1).

One can estimate the fraction of U_3O_8 present in the surface of a UO_2 sample subjected to oxidation from the relative intensities of the characteristic features in the XRD patterns of UO_2/U_3O_7 and U_3O_8 using a procedure recently developed by Choi et al. [37]. According to these authors:

FIGURE 8/1: XRD Patterns of UO₂ Disk Surfaces Subjected to Gamma Fields at 150°C in: (a) Vials Open to the Atmosphere of the Vessel (Experiment A, Sample M); (b) Vials Sealed with Air (Experiment B, Sample L)

FIGURE 8/2: XRD Patterns of UO₂ Disk Surfaces Subjected to Gamma Fields at 150°C in: (c) Sealed Vials, O_2 + 60% Saturated Steam (Experiment C, Sample X) and; (d) Vials Sealed with Ar + 60% Saturated Steam (Experiment D, Sample V)

FIGURE 9/1: XRD Patterns, for 20 Region Between 15-40° of UO₂ Disk Surfaces in: (a) Vials Open to the Atmosphere of the Vessel (Experiment A, Sample M); (b) Vials Sealed with Air (Experiment B, Sample L)

FIGURE 9/2: XRD Patterns, for 20 Region Between 15-40° of UO₂ Disk Surfaces in: (c) Sealed Vials, O_2 + 60% Saturated Steam (Experiment C, Sample X) and; (d) Vials Sealed with Ar + 60% Saturated Steam (Experiment D, Sample V)

FIGURE 10: XRD Patterns, for 20 Region Between 15-40° of UO₂ Disk Surfaces in Vials Sealed with Ar + 60% Saturated Steam (Experiment D): (a) Sample V and (b) Sample O (with O_2 Impurity?)

TABLE 2

COMPARISON OF XPS AND XRD ANALYSIS OF

THE SURFACE OXIDATION OF UO, DISKS AT 150°C IN GAMMA FIELDS

* See Footnotes under Table 1.

^a The phases are listed in order of decreasing intensity in the XRD pattern.

$$
F = I_1/(I_1 + \alpha I_o) \tag{1}
$$

where

 \bar{z}

- the integrated intensity of I11 features of UO_2/U_3O_7 (d ~0.31 mm and 20 $I_0 =$ -28.5°),
- the integrated intensity of 130 and 200 features of U_3O_8 (d ~0.34 mm and 20 $I_1 =$ -26°),
- $\alpha =$ an empirical factor determined by Choi et al. with a value of 0.4504, and
- $F =$ the fraction of U_3O_8 in the XRD sampling depth (~1 to 3 µm)

Table 3 lists values of I_0 , I_1 and F for all the samples investigated using XRD. Values of F are not included for samples W and X because, as noted above, the major surface phases were UO, hydrates. Major peaks from these phases overlap the 0.34 nm feature for U_3O_8 , so that the calculation of F has no meaning in this situation.

The calculations of F consistently show that more U_3O_s was formed in the sealed, nominally dry air environment (Experiment B) than in unlimited air (Experiment A). They also support the suggestion that vessel Z (Experiment C) was not leak-tight, and hence behaved like a Type A Experiment. Overall, these XRD results are consistent with the XPS findings.

The samples exposed to H,0 and Ar atmosphere (samples V and O, Experiment D) show no sign of U_3O_8 formation (Figure 10, Table 3). Figures 10 and 11 compare the XRD patterns of these two samples for the 26 regions from 15 to 40° and 55 to 60°, respectively. Both samples were nominally heated in $Ar + 60\%$ ss environment. The XPS results indicate slight oxidation of the surface of sample 0 (Table 1). The XRD pattern of sample V is identical to that expected for pure UO, [29,31,35], whereas the pattern of sample O shows the presence of small amounts of U_4O_9/U_3O_7 phases in the surface, as indicated by the appearance of weak shoulders on the high-angle side of the 200 (d ~0.273 nm, 2 θ ~32.8°), 311 (d ~0.165 nm, 29 -55.8°) and 222 *(ti* -0.157 nm, 29 -58.5°) peaks of uraninit e [30,33], Figures 10 and 11.

3.4 SCANNING ELECTRON MICROSCOPY

The surface morphology of the samples, as seen in the SEM micrographs, is shown in Figures 12 and 13 (these micrographs are for the same samples whose XRD patterns are shown in Figures 8 and 9). Figure 13 gives a view at higher magnification of the samples. The surface of a sample exposed to the H₂O + Ar atmosphere (sample V, Experiment D), Figures 12d and 13d, is typical of that seen for freshly polished, i.e., unoxidized $\mathsf{U}\mathsf{O}_2$. In contrast, the surface of a sample exposed to an atmosphere of $H_o + O_o$ is covered with crystals with a distinctly different morphology from the underlying matrix, indicating a surface recrystallization process, Figures 12c and 13c (sample X, Experiment C). This is consistent with the XPS and XRD results which indicate that this sample had undergone surface oxidation to hydrated $UO₃$ phases. The samples exposed to air (Experiments A and B) show some signs of surface alteration due to oxidation, Figures 12a, 12b, 13a and Lib.

3.5 DISCUSSION

The results presented here show that measurable quantities of U_3O_8 are formed on UO_2 disks exposed to air or O_2 at 150°C in the presence of gamma fields, comparable to those associated with $10-$ to 20-year old used fuel, in -2.1 a. According to the information available, this is the first definitive observation of UO₂ oxidation by air or O₂ to U₃O₈ at such a low temperature. The formation of U_3O_8 by air or O_2 has been observed only in studies carried out at temperatures $\geq 200^{\circ}$ C [13,27,35], most of which were carried out without any applied gamma fields. (Note that traces of U_3O_s have been detected in the most recent examination of used CANDU fuel from the CEX-1 Experiment - irradiated $\rm UO_{2}$ heated in air at 150° C for \sim 12 a [38].)

FIGURE 11: XRD Patterns, for 20 Region Between 54-60°, of UO₂ Disk Surfaces in Vials Sealed with Ar + 60% Saturated Steam (Experiment D): (a) Sample V; and (b) Sample O (with O₂ Impurity?)

FIGURE 12: Lower Magnification SEM Micrographs of UO₂ Disk Surfaces in: (a) Vials Open to the Atmosphere of the Vessel (Experiment A, Sample M); (b) Vials
Seals with Air (Experiment B, Sample L);; (c) Sealed Vials, $O_2 + 60\%$ Saturated Steam (Experiment C, Sample X); and (d) Vials Sealed with Ar + 60% Saturated Steam (Experiment D, Sample V)

FIGURE 13: Higher Magnification SEM Micrographs of UO₂ Disk Surfaces in: (a) Vials Open to the Atmosphere of the Vessel (Experiment A, Sample M); (b) Vials Seals with Air (Experiment B, Sample L);; (c) Sealed Vials, $O_2 + 60\%$ Saturated Steam (Experiment C, Sample X); and (d) Vials Sealed with Ar + 60% Saturated Steam (Experiment D, Sample V). These samples are the same as used in Figure 12.

CALCULATION OF FRACTION OF U₃O_x FORMATION FROM XRD

* See footnotes under Table 1.

 $I_0 =$ Intensity of 111 features of UO_2/U_3O_7 .

 I_i = Intensity of 130/200 features of U_3O_s .

 $F =$ Fraction of U_1O_s in surface calculated using Equation (1), see text.

 $\dot{\phi}$ = Not calculated due to interference from UO₃*xH,O phases.

Recently Taylor et al. have reviewed the literature on U_3O_8 formation by high temperature air oxidation of UO₂ and estimated times to reach three stages of formation of U_3O_8 at temperatures below 200^oC [13]. The three stages of U_3O_s formation refer to the amount of UO, oxidized to U_3O_8 . The first stage refers to the formation of a detectable (by XRD) quantity of U_3O_s , the second stage refers to the formation of U_3O_s as a minor surface phase (i.e., about 4 to 20% U_3O_8 in the surface layer probed by XRD), and the third stage refers to visible powder formation caused by oxidation of UO_2 to U_3O_8 . They estimated the time needed to reach the first stage of U_3O_8 formation at 150°C as 1.9 a, which is close to 2.1 a, used in the present work (by coincidence the actual period at 150° C is ~1.9 a if one removes the cooling period from the heating-cooling cycle (see Section 2.1)). Also, the amount of U_3O_8 formed in Experiment A (open vials) is close to that expected for stage 1 of U_3O_8 formation, as described by Taylor et al. [13], Table 3.

Although U_3O_8 formation is seen in all the Experiments involving the heating of UO_2 in air/O, at 150°C in gamma fields (Experiments A, B and C), the amounts of U_3O_8 formed in the three Experiments are significantly different. Here we briefly discuss the possible causes

of these differences. The greater oxidation of UO, disks observed in closed air vials (Experiment B) than in open air (Experiment A), Table 1 and 3, at first seems unusual. One would have expected greater oxidation in samples of Experiment A, where the O, supply (from the air) was unlimited. The greater oxidation observed in Experiment B compared to that in Experiment A shows that the amount of available O, was not the limiting factor. The temperature, gamma fields and heating time were the same in all Experiments. However, pressure would be higher in the sealed vials at 150°C than in the open vials. (All vials were sealed with gas pressure of 1 atm at room temperature). Therefore, the increased oxidation observed in Experiment B (vials sealed with air) compared with Experiment A (open vials) is probably related to the higher partial pressure of oxidant(s) in the sealed vials.

It has been shown that the rate of $UO₂$ oxidation is essentially independent of $O₂$ pressure under the partial pressures of $O₂$ used in Experiments A and B [35,39,40]. Thus, it is unlikely that the increased oxidation observed in Experiment B was due to higher partial pressure of O, in the sealed vials. Therefore, we believe that the increased oxidation observed in Experiment B (vials sealed with air) compared with the Experiment A (open vials with air) is most probably due to the higher concentration of radiolysis products of water and air in Experiment B than Experiment A. Although no water was deliberately added to the vials used in Experiment B, there would have been some adsorbed H_o on the UO₂ surfaces [6] and on the glass walls at room temperature, in addition to the water present in the ambient air. It is well known that the radical oxidants formed during radiolysis of water (e.g., OH and O_2) can strongly enhance the oxidation of UO_2 [41]. The oxidation rates of UO_2 by the radical oxidants are several orders of magnitude higher than that by molecular $O₂$ [42].

Radiolysis of air leads to the formation of nitrogen oxides, NO_x , which can also participate in $UO₂$ oxidation [43]. The NO_x formed in open system (Experiment A) can diffuse out, in addition to reacting with the $UO₂$ surface. In a closed system, however, NO_x cannot diffuse out and hence is available for reaction with $UO₂$. This may also have contributed to the higher oxidation observed in Experiment B compared with Experiment A (Tables 1 and 3).

Our suggestion that the main reason for the higher oxidation observed in Experiment B compared with Experiment A is the higher concentrations of water and air radiolysis products is supported by the work of Campbell et al. [43] and Einziger et al. [44], who reported increased oxidation of UO, and PWR fuel in the presence of moisture (with gamma fields) and nitrogen oxides. On the other hand, H_2O radiolysis products in the absence of O_2 , Experiment D (Ar + 60% ss), do not seem to cause any oxidation of $UO₂$ at 150°C. The radiolysis of water produces both oxidants and reductants, e.g.,

$$
H_2O \to OH + H \tag{2}
$$

(the OH radical is a potential oxidant, while H is a reductant). At room temperature, the radiolysis of water has been shown to cause $UO₂$ oxidation, even in the absence of $O₂$. For example, UO₂ oxidation has been observed in Ar-purged solutions undergoing radiolysis [41] due to the relatively higher reactivity of oxidants compared to that of reductants at room temperature. At higher temperatures, i.e., ~150°C, the increased reactivity of the reductants

The UO₂ samples in Experiment C ($O₂ + 60%$ ss) appear to have undergone the most extensive oxidation. Surfaces of these samples are covered with UO_3 xH, O phases in addition to U_3O_8 phase(s), as discussed earlier (Sections 3.3 and 3.4). Taylor et al. observed the formation of similar UO₃*xH₂O and U₃O_s phases at ~225^oC during air oxidation of unirradiated $UO₂$ under high humidity conditions [11,12,14]. They attributed the formation of these phases to the surface crystallization of UO, oxidation products in a "liquid-like" adsorbed water layer on UO₁. Also, XPS studies have shown that a surface layer with a U⁶⁺/U⁴⁺ ratio of ~2, i.e., that expected for U₃O₈, is formed on UO₂ electrodes at room temperature during potentiostatic oxidation of UO₂ at higher potentials [22]. Experiments have been started, under varying humidity conditions, to delineate the relative roles of radiolysis and the adsorbed water layer in the formation of U_3O_8 and UO_3 xH₃O during oxidation of $UO₂$ at ~150°C.

Campbell et al. [43] have suggested that the radiolysis products of N_2 in air (e.g., NO₂) can increase the oxidation rate of UO₂. The increased oxidation in Experiment C (O₂ + 60% ss), which contained no N_2 , compared with that in Experiments A and B, which contained both N_2 and O₂, suggests that the radiolysis products of H₂O are probably much more significant than the radiolysis products of N_2 in terms of UO_2 oxidation. Further Experiments are planned to resolve the role of nitrogen oxides in UO, oxidation.

The results presented above show that U_3O_8 and higher oxides can form during UO_2 oxidation by air at 150°C in gamma fields similar to those associated with 10- to 20-year-old used CANDU fuel. The observation of traces of U_3O_8 in the most recent examination of CEX-1 fuel is consistent with this conclusion. Garisto [7] and, more recently, Kolar [46] have modelled the first stage in the oxidation of used fuel, i.e.,

$$
3UO_2 + \frac{1}{2}O_2 \rightarrow U_3O_7 \tag{3}
$$

The results presented here suggest that the Reaction

$$
3\text{UO}_2 + \text{O}_2 \rightarrow \text{U}_3\text{O}_8 \tag{4}
$$

can occur at 150°C in measurable amounts in the presence of gamma radiation.

Also, the Reaction

$$
UO_2 + \frac{1}{2}O_2 \rightarrow UO_3 \tag{5}
$$

can occur at 150°C in the presence of gamma radiation if both O_2 and H_2O are present. Therefore, we believe that any attempt to model the oxidation of used $UO₂$ fuel at 150°C by air and moist air should include Reactions (4) and (5).

The results presented here and those discussed by Campbell et al. [43] suggest that gamma radiolysis accelerates the oxidation of UO, by air and moist air. The gamma fields near the surface of a used fuel sample are a function not only of its burnup and cooling time but also of the amount of used fuel in the sample (and sample geometry). Thus, the dose rate near the surface of a used nuclear fuel sample consisting of a few grams will be much smaller than that near the surface of a fuel sample consisting of several kilograms (e.g., a complete fuel bundle). Therefore, we believe that the results of the laboratory Experiments, carried out using small fuel samples (i.e., a few grams), would underestimate the effects of gamma radiolysis on UO₂ oxidation for situations involving larger samples (e.g., several fuel bundles) unless external gamma fields are applied.

Although we are able to detect the formation of U_3O_8 on UO, samples exposed to air and gamma radiation at 150°C, the rate of growth of U_3O_s in the absence of moisture (Experiment A) is quite slow. Thus, according to the calculated values in Table 3, the fraction of U_3O_8 present in the surface layer probed by XRD analysis is less than 0.7%. This represents a very slow rate of UO_2 oxidation to U_3O_8 .

4. SUMMARY AND CONCLUSIONS

Oxidation of UO_2 at 150°C by air or O_2 in the presence of gamma fields equivalent to those associated with 10- to 20-year-old used CANDU fuel can lead to the formation of U_3O_8 on a UO₂ surface. The rate of formation of U₂O₈ on UO₂ in dry air at 150°C and gamma fields of ~15 Gy/h is very low. Oxidation of UO₂ to U₃O₈ (and UO₃•xH₂O) by air or O₂ in the presence of gamma radiation is strongly enhanced by the presence of water vapour. However, water vapour radiolysis, in the absence of O_2 (or other oxidizing agents), does not cause UO_2 oxidation at 150°C.

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REFERENCES

- 1. D.G. Boase and T.T. Vandergraaf. 1977. The Canadian Spent Fuel Storage Canister: Some Materials Aspects, Nucl. Technol. 32. 60.
- 2. I.E. Oldaker. 1979. The WNRE Program to Investigate the Stability of Irradiated CANDU Power-Reactor Fuel under Dry Storage Conditions, Atomic Energy of Canada Limited Report, AECL-6431.
- 3. K.M. Wasywich, J.D. Chen, C.R. Frost and J. Freire-Canosa. 1984. Long-Term Behaviour of Irradiated CANDU Fuel in Concrete Canister Storage - Test Results. In Irradiated Fuel Storage: Operating Experience and Development Programs, Toronto, Ontario, 1984, 393.
- 4. K.M. Wasywich and C.R. Frost. 1988. Canadian Experience with the Dry Storage of Used CANDU Fuel. In the Proceedings of the CNA Annual Conference at Winnipeg, Manitoba, June 12-15.
- 5. K.M. Wasywich, W.H. Hocking, D.W. Shoesmith and P. Taylor. 1993. Differences in Oxidation Behaviour of Used CANDU Fuel During Prolonged Storage in Moisture-Saturated Air and Dry Air at 150°C. Nucl. Tech. 104, 309.
- 6. P. Taylor, D.D. Wood, A.M. Duclos and D.G. Owen. 1989. Formation of Uranium Trioxide Hydrates on UO, Fuel in Air-Steam Mixtures Near 200°C. J. Nucl. Mater. 168. 70.
- 7. F. Garisto. 1993. Modelling the Oxidation of Defected Fuel Elements. Atomic Energy of Canada Limited Report, AECL-10734, COG-92-311.
- 8. H.J. Smith, J.C. Tait and R.E. Von Massow. 1987. Radioactive Decay Properties of Bruce "A" $\mathsf{CANDU}^{\mathsf{TM}}$ UO_2 Fuel and Recycle Waste. Atomic Energy of Canada Limited Report, AECL-9072.
- 9. W.H. Hocking, A.M. Duclos and L.H. Johnson. 1994. Study of Fission Product Segregation in Used CANDU Fuel by X-ray Photoelectron Spectroscopy (XPS) II. J. Nucl. Mater. 209. 1.
- 10. P. Taylor, E.A. Burgess and D.G. Owen. 1980. An X-ray Diffraction Study of the Formation of β -UO_{2,33} on UO₂ Pellet Surfaces in Air at 229 to 275°C. J. Nucl. Mater. 88, 153.
- 11. P. Taylor, D.D. Wood, D.G. Owen and G.-I. Park. 1991. Crystallization of U_3O_8 and Hydrated UO_3 on UO_2 Fuel in Aerated Water near 200°C. J. Nucl. Mater. 183, 105.
- 12. P. Taylor, D.D. Wood, D.G. Owen, W.G. Hutchings and A.M. Duclos. 1991. Microstructures and Phase Relationships of Crystalline Oxidation Products Formed on Unused CANDU Fuel Exposed to Aerated Steam and Aerated Water near 200°C. Atomic Energy of Canada Limited Report, AECL-10476.
- 13. P. Taylor, D.D. Wood and A.M. Duclos. 1992. The Early Stages of U_3O_8 Formation on Fresh CANDU Fuel Oxidized in Air at 200-300°C. J. Nucl. Mater. 189. 116.
- 14. P. Taylor, R.L. Lemire and D.W. Wood. 1993 The Influence of Moisture on Air Oxidation of UO₂: Calculations and Observations. Nucl. Technol. 104, 164.
- 15. S. Sunder, G.D. Boyer and N.H. Miller. 1990. XPS Studies of UO, Oxidation by Alpha Radiolysis of Water at 100°C. J. Nucl. Mater. 175. 163.
- 16. Cullen, M.G. 1981. CEGB Steam Tables (Revised). Central Electricity Generating Board, London, U.K. Report # CC/N865, pp. 1-18.
- 17. M.A. Ryz. 1983. The Whiteshell Nuclear Research Establishment Immobilized Test Facility. Proceedings of the 31st Conference on Remote Systems Technology, Publ. Am. Nucl. Soc. pp 140-148.
- 18. S. Sunder, N.H. Miller and A.M. Duclos. 1994. XPS and XRD Studies of Samples from the Natural Fission Reactors in the Oklo Uranium Deposits. Mater. Res. Soc. Symp. Proc. 333 (Scientific Basis for Nuclear Waste Management XVII), pp. 631-8.
- 19. C.D. Wagner, W.M. Riggs, L.E. Davis, J.F. Moulder and G.E. Muilenberg. 1979. Handbook of X-ray Photoelectron Spectroscopy, Perkin-Elmer, Eden Prairie, Minnesota.
- 20. N.S. Mclntyre, S. Sunder, D.W. Shoesmith and F.W. Stanchell. 1981. Chemical Information from XPS - Applications to the Analysis of Electrode Surfaces. J. Vac. Sci. Technol. 18, 714.
- 21. D. Briggs and M.P. Seah, Eds. 1983. Practical Surface Analysis by Auger and X-Ray Photoelectron Spectroscopy, J. Wiley & Sons, Chichester.
- 22. S. Sunder, D.W. Shoesmith, M.G. Bailey, F.W. Stanchell and N.S. Mclntyre. 1981. Anodic Oxidation of UO,. Part I, Electrochemical and X-ray Photoelectron Spectroscopic Studies in Neutral Solutions. J. Electroanal. Chem. 130, 163.
- 23. J. Verbist, J. Riga, C. Tenret-Noel, J.J. Pireaux, G. d'Ursel, R. Caudano and E.G. Derouane. 1976. Bonding and Valence in Uranium Compounds with Oxygen: A Study by X-ray Photoelectron Spectroscopy and Electron Paramagnetic Resonance. In Plutonium and other Actinides, eds. H. Blank and R. Lindner. North-Holland, Amsterdam, pp. 409-419.
- 24. B.W. Veal and D.J. Lam. 1982. Photoemission Spectra in Gmelin Handbook of Inorganic Chemistry Suppl. Vol. A5. Springer Verlag, Heidelberg pp. 176-196.
- 25. G.C. Allan, P.A. Tempest and J.W. Tyler. 1987. J. Chem. Soc. Faraday Trans. I., 83, 925.
- 26. G.C. Allen and N.R. Holmes. 1987. Surface Characterisation of α -, β -, γ -, and δ -UO₃ Using X-Ray Photoelectron Spectroscopy. J. Chem. Soc. Dalton Trans., 3009.
- 27. G.C. Allen, P.A. Tempest and J.W. Tyler. 1987. Oxidation of Crystalline UO, Studied Using X-ray Photoelectron Spectroscopy and X-ray Diffraction. J. Chem. Soc. Faraday Trans. 1, 83, 925.
- 28. S. Sunder, J.J. Cramer and N.H. Miller. 1992. X-ray Photoelectron Spectroscopic Study of Cigar Lake Uranium Ore: A Natural Analog for Used Fuel. Mater. Res. Soc. Symp. Proc. 257, Scientific Basis for Nuclear Waste Management, XV, pp. 449-457.
- 29. H.R. Hoekstra, A. Santoro and S. Siegel. 1961. The Low Temperature Oxidation of UO2 and *\J.O9.* J. Inorg. Nucl. Chem. 18 166.
- 30. O.K. Smith, B.E. Scheetz, C.A.F. Anderson and K.L. Smith. 1982. Phase Relations in the Uranium-Oxygen-Water System and Its Significance on the Stability of Nuclear Waste Forms. Uranium, 1, 79.
- 31. D. Vollath. 1984. In Uranium-Uranium Dioxide, UO₂, Preparation and Crystallographic Properties. Gmelin Handbook of Inorganic Chemistry, Vol. C4, Springer-Verlag, Heidelberg, p. 97.
- 32. J. Janeczek, and R.C. Ewing. 1991. X-ray Powder Diffraction Study of Annealed Uraninite. J. Nucl. Mater. 185. 66.
- 33. L.E. Thomas, R.E. Einziger and H.C. Buchanan. 1993. Effect of Fission Products on Air-Oxidation of LWR Spent Fuel. J. Nucl. Mater. 201. 310.
- 34. B.T.Willis. 1987. Crystallographic Studies of Anion-excess Uranium Oxides. J. Chem. Soc. 83, 1073-1081.
- 35. P.A. Tempest, P.N. Tucker and J.W. Tyler, 1988. Oxidation of $UO₂$ Fuel Pellets in Air at 503 and 543 K Studied Using X-ray Photoelectron Spectroscopy and X-ray Diffraction. J. Nucl. Mater. 185, 251.
- 36. Powder Diffraction Files. 1994. Inorganic Materials, Inter. Center for Diffraction Data, Newtown Square, Penn.
- 37. J.W. Choi, R. McEachern and P. Taylor, unpublished work.
- 38. P. Taylor, Unpublished work.
- 39. P.M. Tucker. 1987. The Effect of Oxygen Partial Pressure on the Kinetics of Unirradiated $UO₂$ Oxidation. In Chemical Reactivity of Oxide Fuel and Fission Product Release. Vol. 1, K.A. Simpson and P. Wood (eds.), Central Electricity Generating Board, London, U.K.
- 40. J. Nakamura, T. Otomo and S. Kawasaki. 1993. Oxidation of UO₂ Under Dry Storage Conditions. J. Nucl. Sc. Tech. 30, 181.
- 41. S. Sunder, D.W. Shoesmith, H. Christensen, N.H. Miller and M.G. Bailey. 1990. Oxidation of U02 Fuel by Radicals Formed During Radiolysis of Water. Mater. Res. Soc. Symp. Proc., 176, (Scientific Basis for Nuclear Waste Management XIII), pp. 457- 464.
- 42. H. Christensen, S. Sunder and D.W. Shoesmith. 1994. Oxidation of Nuclear Fuel UO₂ by the Products of Water Radiolysis Development of a Kinetic Model. J. Alloys and Compounds. 213/214. 93.
- 43. T.K. Campbell, E.R. Gilbert, G.D. White, G.F. Piepel and B.J. Wrona. 1989. Oxidation Behaviour of Nonirradiated $UO₂$. Nucl. Technol. 85 , 160.
- 44. R.E. Einziger, S.C. Marschman and H.C. Buchanan. 1991. Spent Fuel Dry-Bath Oxidation Testing. Nucl. Technol. 94, 383.
- 45. G.V. Buxton, C.L. Greenstock, W.P. Helmand and A.B. Ross. 1988. Critical Review of Rate Constants for Reactions of Hydrated Electrons, Hydrogen Atoms and Hydroxyl Radicals in Aqueous Solution. J. Phys. Chem. Ref. Data, 17, 513.
- 46. M. Kolar. 1994. Application of the Integral Method to Modelling the Oxidation of Defected Fuel Elements. Atomic Energy of Canada Limited Report, AECL-11174, COG-1-94-446.

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APPENDIX A

DOSE RATE IN THE EXPERIMENT

 $\sim 10^{-11}$

 $\mathcal{L}^{\text{max}}_{\text{max}}$, where $\mathcal{L}^{\text{max}}_{\text{max}}$

 $\sim 10^{11}$ km $^{-1}$

APPENDIX A

DOSE RATE IN THE EXPERIMENT

The used fuel samples surrounding the Experiments described here were part of the Experiment called ACX-1, alternate controlled environment Experiment, also called CEX-2 (Sections 1 and 2.1 of main text). To estimate gamma dose rate experienced by the $UO₂$ samples used in present study the dose rate measurements were carried out at the beginning of the Experiment ACX-2 as the gamma dose rate during these Experiments (and ACX-1) was not measured. ACX-2 uses the same Experimental set-up, but contains one less fuel rod in each of the four fuel vessels, Figure 3. (These fuel rods were removed for examination). Thus, the total number of fuel rods in the ACX-2 Experiment is 48 instead of 52 in the ACX-1. The dose rate was measured, using a Curie Pie Model CPMU # 01743 obtained from Technical Associate Canoga Park, CA. U.S.A., at two points: (a) at the top of the aluminum vessel (T in Figure A-1), $R_T = 3.20$ Gy/h; and (b) and at a point 15 cm above the top of the aluminum-vessel (T' in Figure A-1), $R_T = 1.40$ Gy/h. Here R_T and R_T are measured dose rates at points T and T'.

In order to calculate gamma dose rate experienced by the $UO₂$ samples from the above measured values, we approximate the used fuel rods surrounding the aluminum vessel with a point source located at a distance equal to the distance between the centre axes of the fuel vessels and of the aluminum sample vessel (A and B, respectively, in Figures 3 and A-l); and at a height equal to the middle of the aluminum vessel (A in Figure A-l). With these approximations the dose rate at the centre of the aluminum vessel (B in Figure A-1), R_B is:

$$
R_B = R_T \cdot (AT^2/AB^2)
$$

= R_T \cdot [(AB^2 + BT^2)/AB^2]
= 3.20 \cdot [(15.6^2 + 31^2)/15.6^2] Gy/h
- 15 Gy/h

Here AB and BT are 15.6 and 31 cm, respectively.

If we use the same procedure to calculate the dose rate at T' from the measured value at T $(TT = 15 \text{ cm})$, i.e.:

$$
R_T = R_T \cdot (AT^2/AT^2)
$$

= R_T \cdot (AT^2/(AB + TT^1)^2)
= 3.20 \cdot ((15.6^2 + 31^2)/(15.6^2 + (31 + 15.0)^2)) Gy/h
- 1.6 Gy/h

This compares favourably with the measured value of R_T , i.e., 1.4 Gy/h and gives confidence in the assumptions made in the above calculations. Thus, the dose rate in the aluminum vessel varies along the vertical axis between 15 and 3.4 Gy/h. These dose rate are similar to those expected on the surface of 10 to 20 a old CANDU fuel [Al]. The dose rate in the

aluminum vessel would decrease with an increase in the distance from the middle point. However, the dose rate would increase with an increase in the distance from the centre of the aluminum vessel, i.e., the points closer to the used fuel (away from the centre axis of aluminum vessel) would experience higher gamma fields. We believe that the dose rate experienced by the UO₂ samples was closer to 15 Gy/h during the Experiment as the samples were closer to the middle of the aluminum vessel.

REFERENCE

A.1 D.W. Shoesmith and S. Sunder. 1992. The Prediction of Nuclear Fuel (UO₂) Dissolution Rates Under Waste Disposal Conditions. J. Nucl. Mater. 190, 20.

FIGURE A-l: Schematic of Locations Used for Calculating Dose Rate: A, Centre of Fuel Vessel; B, Centre of Aluminum Vessel Containing UO₂ Samples (see Figure 3); and T, Top of the Aluminum Vessel. T and T are Locations of Dose Rate Measurements (see text).

J.

APPENDIX B

 \sim \sim

 $\mathcal{A}^{\mathcal{A}}$

 $\bar{\mathcal{A}}$

APPENDIX B

TABLE B-l

XRD DATA FOR UO, OXIDATION AT 150°C IN GAMMA RADIATION IN OPEN AIR

(Sample M. Experiment A)

 $\hat{\mathcal{L}}$

continued...

 $\ddot{}$

Intensity values are based on counts per second.

 $\mathcal{A}^{\mathcal{A}}$

 $\mathcal{L}^{\text{max}}_{\text{max}}$

TABLE B-2

XRD DATA FOR UO, OXIDATION AT 150°C IN GAMMA RADIATION IN

CLOSED AIR

(Sample L, Experiment B)

continued ...

 $\hat{\mathcal{A}}$

* Intensity values are based on counts per second.

TABLE B-3

XRD DATA FOR UO, OXIDATION AT 150°C IN GAMMA RADIATION

continued...

 $\ddot{}$

continued ...

 $\sim 10^6$

Intensity values are based on counts per second.

TABLE B-4

XRD DATA FOR UO, OXIDATION AT 150°C IN GAMMA RADIATION IN (Ar + 60% Saturated Steam: Sample V. Experiment D)

* Intensity values are based on counts per second.

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