



XA9744829

# NUCLEAR FUEL ACTIVITIES IN BELGIUM

presented by Hubert BAIRIOT  
at the IWGFPT Meeting  
Vienna - 21 May 1997

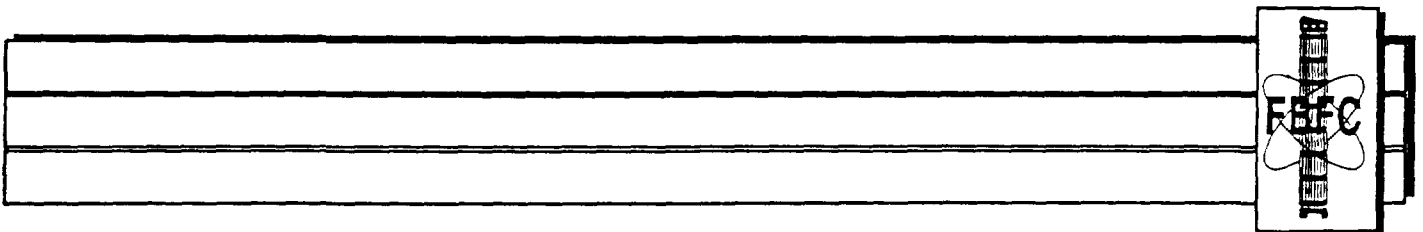
- FUEL FABRICATION
- NPP OPERATION
- FUEL PERFORMANCE
- R&D PROGRAMMES

# FBFC International

**Ownership : 100 % FBFC FRANCE**

**(FBFC FRANCE = 51 % FRAMATOME,  
49 % COGEMA)**

- **Customers** F: EDF  
B : ELECTRABEL/TRACTEBEL  
D : All utilities  
Other : Swedisch and Swiss utilities,...
  
- **Designs currently available**  
Pellets/rods : from 14 x 14 to 18 x 18, UO<sub>2</sub> and  
UO<sub>2</sub>/Gd<sub>2</sub>O<sub>3</sub>  
Fuel assemblies : 8 x 8, 9 x 9, 14 x 14 to 18 x 18, UO<sub>2</sub>  
and MOX
  
- **Safety record**  
From plant start-up (1961) to actual status  
No violation of legal limits on
  - Worker radiation exposure
  - Environmental releases



# FBFC International

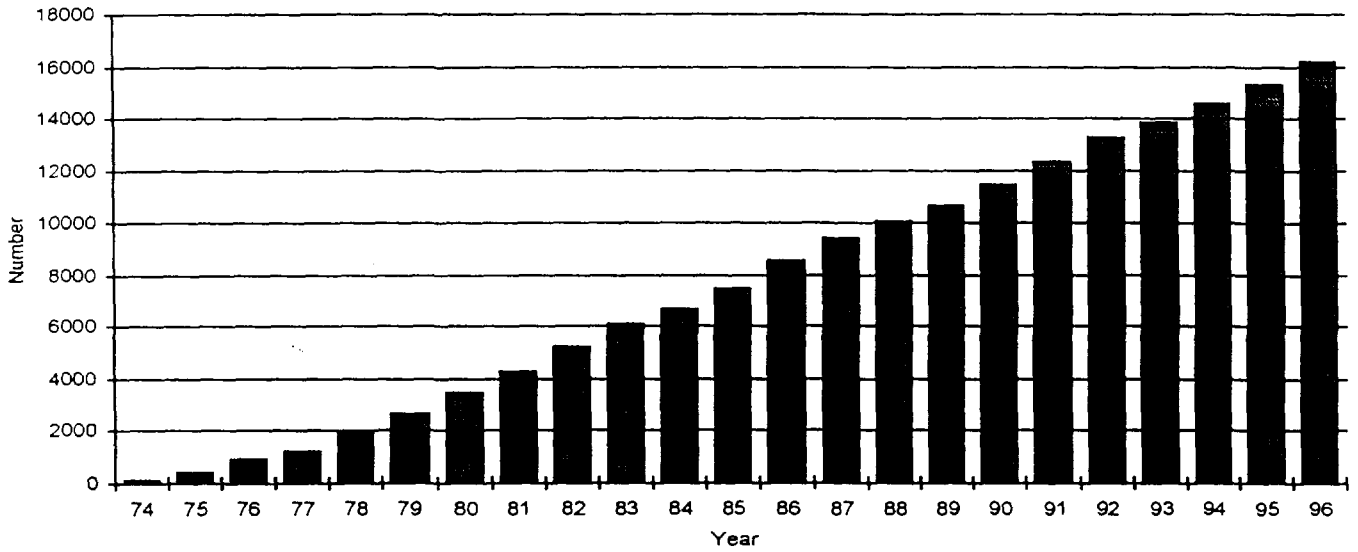
## MAIN RECENT EVENTS

- 1992 : - high fuel assembly load (953 including 104 MOX)  
- first deliveries of pellets to Germany  
- level 2 incident : rupture of a MOX fuel rod
- 1993 : - implementation of lessons learned from MOX incident  
- 557 fuel assemblies including 76 MOX  
- 163 MT U of pellets to Germany  
- introducing new designs (e.g. AFA2G)
- 1994 : - 743 fuel assemblies including 132 MOX  
- new building for MOX assembling under construction (study)  
- realisation of the 2 first BWR 8 x 8 assemblies (prototype for Japan)
- 1995 : - 758 fuel assemblies, including 104 MOX  
- first reload BWR 9 x 9 MOX for Germany  
- 156 fuel assemblies to Siemens PWR designs  
- qualification steps to design 8 x 8 Toshiba
- 1996 : - 828 fuel assemblies, including 88 MOX and 289 Siemens-design  
- MOX-building in operation for storing and packing fuel assemblies  
- safety-improvements



# FBFC International

### CUMULATED PRODUCTION Number of Assemblies (PWR)

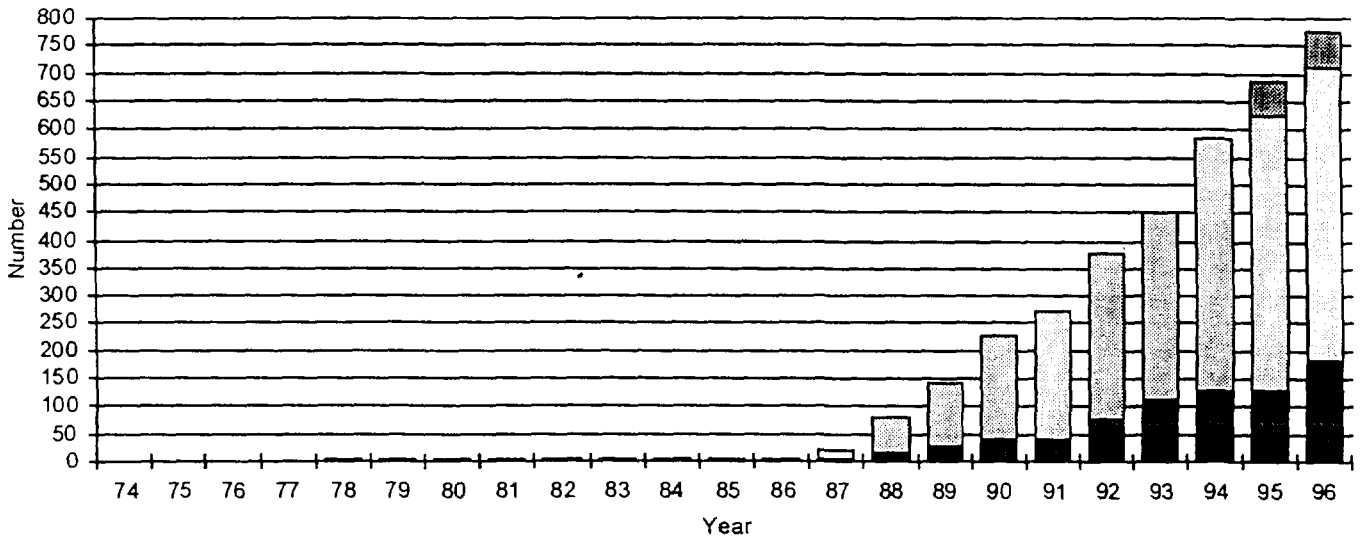


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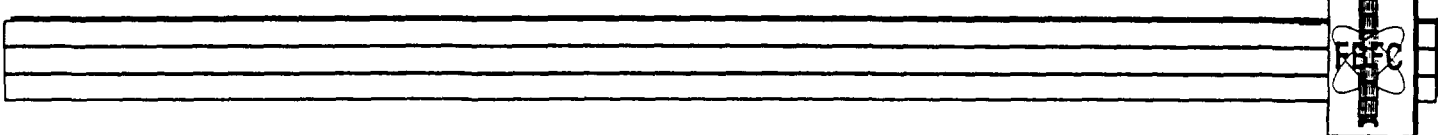


# FBFC International

### Cumulated MOX-Production - Number of Assemblies



88 89





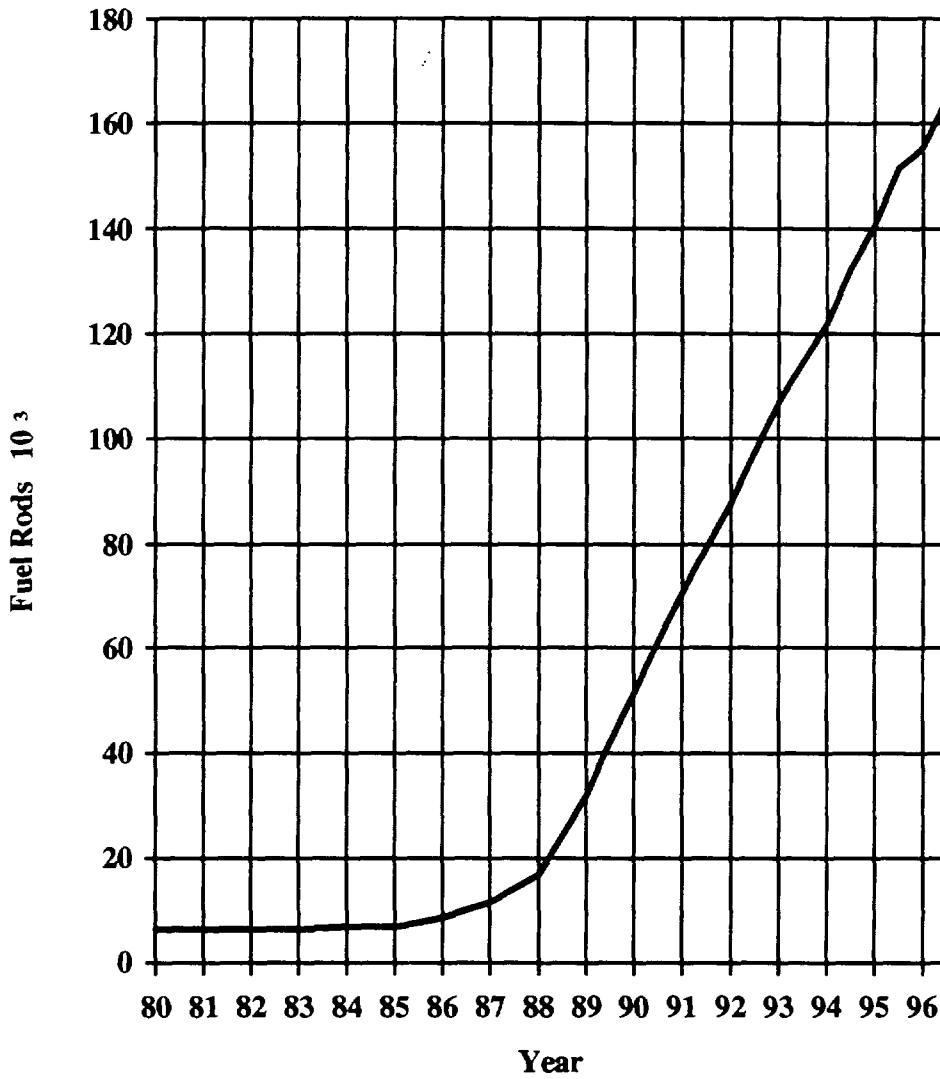
# Plant Performance

	Year	1985	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996
BN Dessel	Capacity (tHM)	4	8	15	25	35	35	35	35	35	35	35	35
	Actual production (tHM)	1	6	15	26	35	37	30	32	36	32	30	36

105

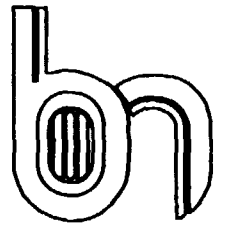


**MOX Fuel Rods from the BN-Dessel Plant  
(cumulated delivered quantities)  
by Oct. 1, 1996**

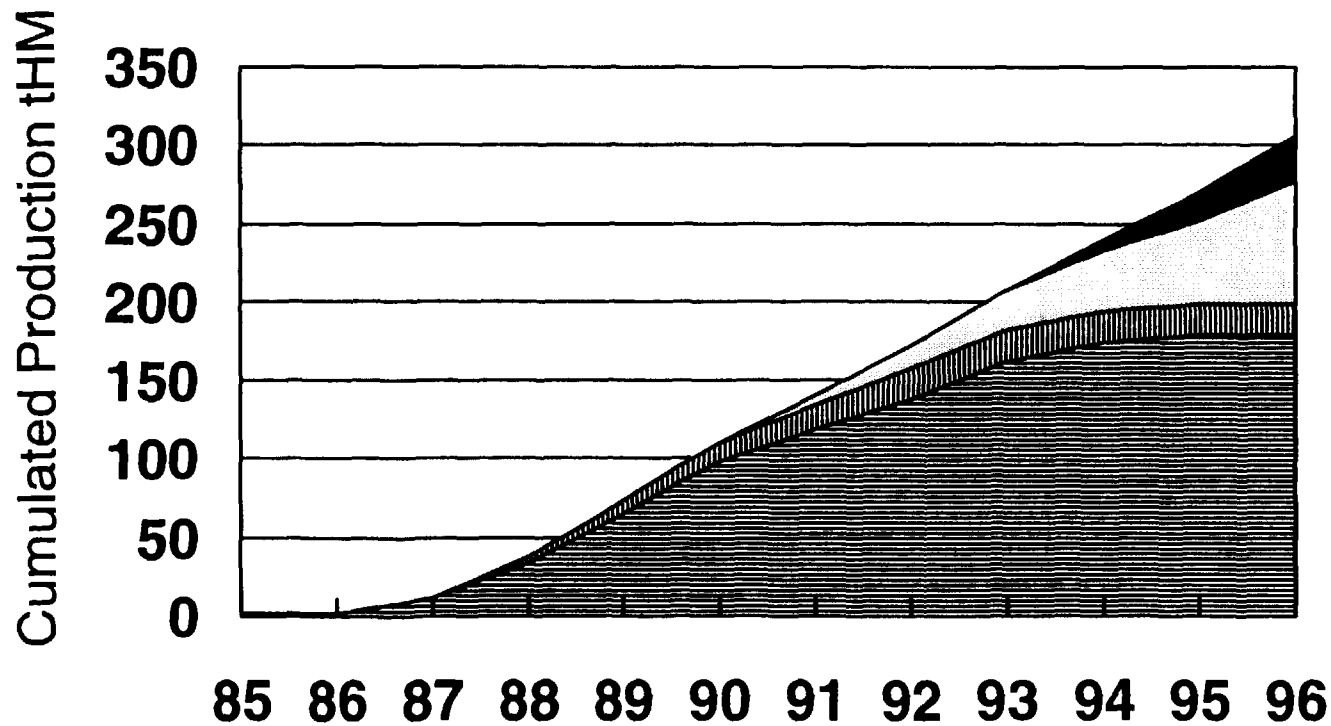


<b>Rods : 164913</b>	<b>159976 MIMAS</b>
<b>tHM : 302.8</b>	<b>294.8</b>
<b>for assemblies :980</b>	<b>(759 with MIMAS)</b>

# Cumulated MIMAS production at BELGONUCLEAIRE



■ France (PWR)   ■ Switzerland (PWR)   □ Germany (PWR + BWR\*)   ■ Belgium (PWR)



Year

101





# OPERATION OF NPPs IN BELGIUM

NPP (PWRs)	KCD1	KCD2	KCD3	KCD4	CNT1	CNT2	CNT3
MW th	1192	1192	3064	3000	2875	2905	3000
UPRATING	-	-	10%	-	8%	4.3%	-
Mwe NET	393	393	1006	1001	931	955	1015
LOAD FACTOR 1996	92%	84%	84%	70%	88%	88%	81%
LAST CYCLE							
BOC - EOC months	10	12	9	10	13	13	14
BOC - BOC months	11	13	11	13	14	15	15
BURNUP Gwd/t	11	12	11	10	15	15	13
AVERAGE LAST 3 CY							
BOC - EOC months	11	11	10	9	14	14	14
BOC - BOC months	12	12	11	12	16	16	15
BURNUP Gwd/t	12	11	12	9	16	16	14
SG REPLACEMENT	-	-	93	96	95	01	98

## FUEL PERFORMANCE IN BELGIAN NPPs

(burnups expressed in GWd/tHM)

NPP (PWRs)	KCD 1	KCD 2	KCD 3	KCD 4	CNT 1	CNT 2	CNT 3
FA/CORE	121	121	157	157	157	157	157
LAST CYCLE							
FA/reload	32	36	44	52	64	52	56
Gd/MOX/RepU	-/-/32	-/-/-	-/8/-	24/-/-	56/-/-	20/12/-	44/-/-
Aver Discharge BU	44	39	44	27	40	47	41
TYPICAL LAST 3 CY							
FA/reload	32	36	44	32	57	52	56
Gd/MOX/RepU	-/-/32	-/-/-	-/8/-	-/-/-	48/-/-	20/8/-	28/-/-
Aver Discharge BU	43	40	44	44	44	46	41
FA vendor							
Present	F	S	S+F	E	F	A+F	F
Previous	id	id	id	F	S	F	id

## FAILED FAs IN BELGIAN NPPs

NPP	ft	1996	94 + 95 + 96
KCD 1	8	-	-
KCD 2	8	1 U	3 U
KCD 3	12	-	1 t
KCD 4	14	1D + 1G + 1U + 1d + 3dg + 8g	1D + 1G + 3U + 1d + 4dg + 11g
CNT 1	12	-	-
CNT 2	12	1 U	1 U
CNT 3	14	1G + 1U + 2g	3G + 2U + 1d + 3 dg + 6g

Leakers	Mechanical failure
D : debris	d : debris
G : grid	dg : debris + grid
	g : grid
U : unknown	t : guide-thimble

### RCCA Insertion Problems in Belgian NPPs

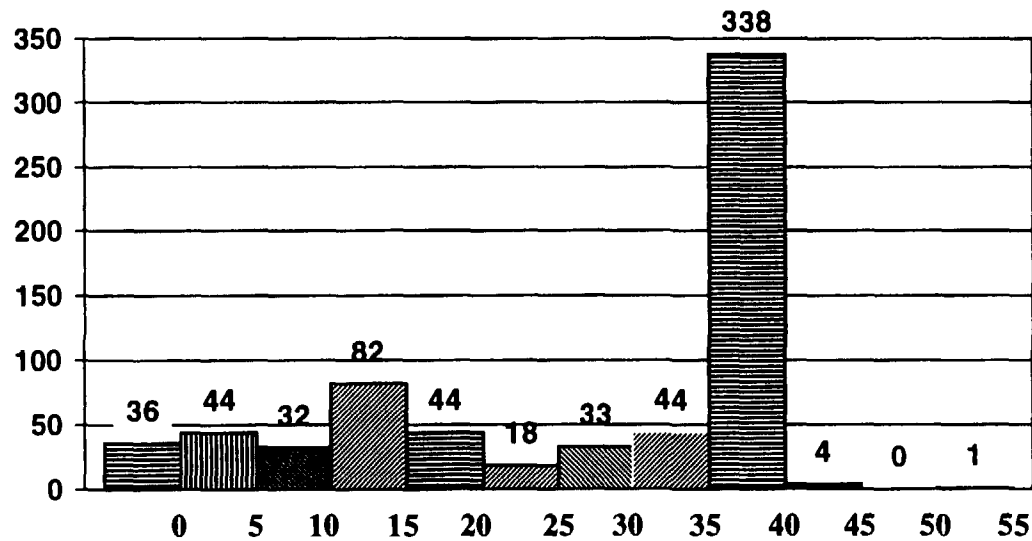
(status 97 05 07)

- Only in 14 ft cores
- KCD4 :
  - cy 11 (9508 - 9608) : 5 blocked in F FAs at EOC
  - cy 12 (9609 - ) : OK
- CNT 3 :
  - cy 10 (9510 - 9611) : 8 blocked in F FAs near EOC
  - cy 11 (9612 - ) : OK

# MIMAS FUEL IRRADIATION EXPERIENCE by Oct. 1, 1996



Number of fuel assemblies



Assembly Burnup (GWd/tHM)

On reactor site  
Not yet start or  
underway to start  
first irradiation cycle

During first  
irradiation cycle

After completed irradiation cycles

Total : 697 FUEL ASSEMBLIES WITH MIMAS  
154630 Fuel Rods

France	: 27 reloads in 8 PWRs
Germany	: 6 reloads in 4 PWRs
	: 2 reloads in 2 BWRs
Switzerland	: 5 reloads in 1 PWR
Belgian	: 4 reloads in 2 PWRs
	+ experimental assemblies in BR3 PWR
The Netherlands	: experimental assemblies in DODEWAARD BWR



**Recent International Programmes  
for  
MOX Fuel Validation and Licensing**

## **1. Neutronic - Core physics validation**

**Experimental investigation** : Critical values, reactivity effects, radial and axial power distribution, spectral indexes, detector responses, source term and actinides, ...

### **VIP - PWR (PWR 17 x 17 MOX assemblies simulation)**

**Facility** : ● VENUS

**Support** : BN, CEN/SCK, MHI (NFI, NEL and Japanese PWR utilities), BNFL.

**Status** : Completed (1990-1993)

### **VIP - BWR (BWR 8 x 8 MOX bundles simulation)**

**Facility** : ● VENUS

**Support** : TOSHIBA, HITACHI ( Japanese BWR utilities), BN, CEN/SCK.

**Status** : Completed (1989-1993)

### **VIPO (Void simulation within high enriched MOX fuel slabs)**

**Facility** : ● VENUS

**Support** : BN, CEN/SCK, EdF, NFI (and Japanese PWR utilities), BNFL, MHI.

**Status** : completed at the end of 1996.



**VIPEX - P** (Extension of VIP-PWR for Am 241 effect, water density effect, control rod worth for MOX versus UO<sub>2</sub>, flux profile within MOX fuel rods, Beta effective, overmoderated configuration, Detector Response)

**Facility :** • VENUS

**Support :** BNFL, KAERI, BN, CEN/SCK, CRIEPI, MHI (Japanese PWR utilities)

**Status :** Start up at mid 1996 (duration 1.5 year)

**ARIANE** (Source term for high B.U. MOX and UO<sub>2</sub> fuel)

**Facilities :** • LWR irradiated commercial fuel from

- Beznau-1 and Gösgen (PWR samples)
- Dodewaard (BWR samples)

• Laboratories : TUI, CEN/SCK, PSI.

**Support :** TOSHIBA, HITACHI (Japanese BWR utilities),  
TRACTEBEL (ELECTRABEL), NOK,  
GKN, PSI, CEN/SCK, JRI, GNB, BN.  
MHI ( NFI, Japanese PWR utilities ) NEL, ORNL, BNFL.

**Status :** Underway since early 1995, duration 3 years including scope extension.



## 2. Rod Thermomechanical behaviour

**Experimental investigation** : Fission gas release, geometry, fuel and clad structure analyses, power ramping, extended PIE etc.

**DOMO** (MOX segments versus UO<sub>2</sub> segments behaviour up to high burnup conditions)

**Facilities** :

- Irradiation in Dodewaard
- PIE and ramps in CEN/SCK
- PIE at PSI.

**Support** : BN, CEN/SCK, TOSHIBA, HITACHI (PNC, NFD, Japanese BWR utilities), PSI, GKN.

**Status** : Under way. (duration : about 10 years)  
End of the programme : early 1997.

**FIGARO** (Instrumentation of MOX PWR irradiated fuel segments for investigating fission gas release and central temperature versus power)

**Facilities** :

- Irradiation in Beznau-1
- PIE + preparation at PSI
- Instrumentation at Kjeller
- Ramp testing at Halden
- PIE after ramp at Kjeller

**Support** : BN, NOK, PSI, MHI, EdF, KAERI, CEN/SCK.  
FRAGEMA (Cogema, Framatome), TRACTEBEL (Electrabel)

**Status** : Starting early 94, duration about 3 years.

**NOK - M109** (Investigation of MOX PWR irradiated fuel rods on fission gas release and PCI)

**Facilities** :

- Irradiation in Beznau-1.
- PIE at PSI .

**Status** : Starting August 1995 (about 2 years)

**Support** : MHI, BNFL, SIEMENS, NOK, BN.





**ZODIAC** An experimental programme to study the  
Zircaloy high dose irradiation creep.

**Objectives**

- to assess the mechanical properties (primary creep) of zircaloy claddings irradiated in power reactors.

**Main features of the programme**

- Zy4 irradiated claddings in Blayais
- Zy2 irradiated claddings in Ringhals.
- Biaxial tensile tests, biaxial creep tests, axial creep tests, relaxation tests, low cycle fatigue tests at temperatures 350-400°C

**Schedule** 1992-1997

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