OPERATIONAL SAFETY FOR A KBS-3H REPOSITORY

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Abstract

In 2001 SKB prepared an R&D programme /1/ for the KBS-3H, horizontal emplacement design of the repository for encapsulated spent nuclear fuel as an alternative repository design to KBS-3V, vertical emplacement. Both are based on the so-called KBS-3 method with multi-barrier systems but the orientation of the canister differs as illustrated in **Figure 1.** It should also be noted that KBS-3V is the reference repository design for spent nuclear fuel in Sweden. The repository for spent fuel is planned to be constructed at a depth of about 500 metres in crystalline bedrock. The total amount of spent fuel within the Swedish programme if the reactors are operated $50 - 60$ years will be in the order of 9,000 tonnes or about 6,000 canisters. The deposition capacity is planned to be in the order of 160 canisters per year. Start-up for emplacement of canisters with encapsulated spent nuclear fuel is now scheduled for 2020 /SKB R&D Report 2007/.

The purpose of this paper is to present preliminary analysis on operational safety for a KBS-3H repository highlighting the two main areas where the KBS-3H and KBS-3V designs differ. The areas are the activities in the reloading station and in the disposal drift

Figure 1: Artist impression of the multi barrier KBS-3 method showing both the vertical deposition (KBS-3V) on the left and the horizontal deposition (KBS-3H) on the right

1 Introduction

In the KBS-3H design, the copper canister with encapsulated spent nuclear fuel and buffer are placed inside a perforated steel cylinder and disposed of as one package, normally called the Super Container (SC). The different components of the SC are transported separately to the reloading station at repository level. The KBS-3H design requires a shielded handling cell in the reloading station for the assembly of the SC. The SC is then transported to the chamber in front of the deposit drift in a "transport tube" that is required for the handling of the SC and radiation shielding. For the KBS-3V design, the copper canister will be reloaded directly from the transport cask into the shielded tube of the deposition machine in the reloading station. The buffer material will be placed in the vertical deposition hole before emplacement of the canister and only the buffer top parts will be installed with the canister in place.

In the KBS-3H design, the SC container is emplaced with a special deposition machine designed for handling heavy loads in to the up to 300 m long horizontal deposition drift. The KBS-3H drift has a diameter of only 1.85m and the SC has a diameter of 1.765m to which the height of the container feet shall be added, about 50 mm. The radial clearances are therefore small and will restrain the handling of the SC in the drifts. The KBS-3H design does not require backfilling of the deposition drifts but installation of distance blocks are needed between the SC for not exceeding the allowed maximum temperatures in the buffer. The thermal load on each canister is limited to 1 700 W for both KBS-3V and KBS-3H. The artist impression of the KBS-3 repository is shown in **Figure 2** and the location of the reloading station is shown on the figure as well as the arrangement of the horizontal drifts.

Figure 2: Artist impression of the KBS-3H repository with the reloading station to the right and some disposal drifts on the left side

Figure 3 shows the main components in the KBS-3H design with the super container which contains the copper canister with encapsulated spent nuclear fuel, the compacted buffer material and the perforated steel shell. **Figure 3** also shows the deposition machine and the installation in the chamber

with transport tube and Start Tube plus the 300m long horizontal disposal drift. The assembly of the SC is done at the repository level in a radiation shielded cell inside the reloading station. The layout of the reloading station is shown in **Figure 4** and **Figure 5.**

The main differences between the KBS-3V and KBS-3H repository designs are in the reloading station and activities associated with emplacement of the SC and distance blocks. This paper is focused on presenting the differences in operation safety or risk analysis for these two areas.

Figure 3: The main components in the KBS-3H concept

2 Reloading Station for KBS-3H Design

The reloading station will have a different function in the KBS-3H design compared to KBS-3V. In KBS-3H the main activities in the reloading station are:

• The pre-assembly of the Super Container is done in the reloading station but outside the shielded handling cell. This pre-assembly will be done inside the transport tube standing vertical on a small wagon in the transport tube pit and with the top gamma gate removed, see **Figure 4** and **Figure 5**. First, the SC perforated shell is placed inside the transport tube and then all buffer material, the top lid of the shell and the transport tube top gamma gate. The complete SC except the spent fuel (SF) copper canister is now ready to be transferred into the transport tube. The top lid of the Super Container is of cause not welded and the top gamma gate is not bolted to the shell of the transport tube at this stage.

International Conference Underground Disposal Unit Design & Emplacement Processes for a Deep Geological Repository. 16-18 June 2008, Prague

Figure 4: Preliminary plan view of the reloading station with the shielded handling cell. The empty transport tube is standing on a wagon in the loading pit but outside the cell. The transport cask for the spent fuel canister standing on its wagon in the loading bit also outside the cell

- As soon as the pre-assembled of the SC is completed and the top gamma gate is in place the transport tube can be transferred with the wagon into the shielded cell for receiving the SF copper canister. In the handling cell, first the top gamma gate is removed as well as the top plate and the top buffer (bentonite) plug, see **Figure 6.**
- The transport cask has been unloaded from the ramp transfer vehicle and the outer protection lid is removed as a preparation for lifting the canister out of the cask. The cask is then placed on a small wagon in the transport cask pit for transport inside the handling cell. Inside the handling cell the inner radiation shielding lid of the transport cask will be removed and the cask is now open.
- The canister can now be lifted out of the cask and placed inside the buffer material in the SC, see left part of **Figure 6**. With the canister in place, the top bentonite block is placed on top of

the canister inside the shell. The top plate of the shell is then installed and welded to the SC shell. All these activities are done inside the transport tube.

- After completion of the welding and control of the results, the top gamma gate can be bolted to the transport tube. The transport tube is now ready for being removed from the handling cell.
- The empty transport cask for the SF canister can now also be prepared for removal including replacing the inner and outer lids of the cask. These operations do not differ from those in the KBS-3V with the exception of the removal of the canister, which is done inside the handling cell in order to provide radiation protection during the transfer of the canister from the cask to the prepared SC in the transport tube.
- The last operation in the reloading station is the transfer of the filled transport tube to the trailer for transport to the chamber in the repository area for emplacement into the deposition drift.

Figure 6: The illustration to the very left shows the removal of the top gamma gate, the SC top lid and the top buffer block. The plan view in the middle just shows the parts that are removed.

The illustration to the very right shows the transfer of the canister from the transport cask to the SC inside the transport tube

Table 1 presents a **"what-if analysis"** performed for the reloading station and the shielded handling cell.

Table 1: What-if analysis for the Reloading station and the shielded handling cell

3 Conclusions Regarding Identified Accident Risks in the Reloading Station and the Handling Cell.

Obviously there are a number of additional risks compared to KBS-3V during the assembly of the SC in the reloading station. But the work in the shielded handling cell must also be compared to the actions where the copper canister has to be lowered into the deposition hole filled with buffer rings in the depository area in the KBS-3V design.

The following additional risks, compared to KBS-3V are identified for this part of the process:

- Lowering of the copper canister into the SC filled with bentonite buffer and lift of transport tube with the SC.
- The canister is moved from the transport cask into the SC inside the transport tube in the shielded handling cell.
- The shielded handling cell contains equipment that may cause additional risks, e.g. increase the fire ignition probability.

The risk in the transport of the transport cask with the canister in the ramp is the same as for the KBS-3V design and therefore not discussed in the paper.

4 Identified Operational Accident Risks in the Depository Area and in the Drift

The transport of the loaded transport tube to the chamber in the deposition area is similar to the transport of the canister inside the shielded tube in the deposition machine for the KBS-3V design.

However, the installation of the SC and distance blocks inside the 300m long drift with diameter of 1.85m is shown on **Figure 7** differs a lot between KBS-3V and KBS-3H**.** The equipment needed for the handling and emplacement of the SC into the deposition drift is presented in a separate paper in this conference and is therefore not presented here.

Figure 7: Artist impression of the KBS-3H deposition drift

When the transport tube with the SC has been transported to the repository area, the following activities are performed:

- placing the transport frame with the transport tube in the correct position in the chamber in front of the deposition drift;
- docking of the transport tube to the deposition drift;
- docking of the deposit machine to the transport tube;
- initialization of the deposit process for the SC;
- transportation of the SC inside the deposition drift;
- emplacement of the SC in the deposition drift;
- transport and emplacement of distance blocks in the drift;
- if needed also fixing rings and compartment plugs may be installed depending of the inflow of water into the deposition drift. Post-grouting for reduction of ground water inflow into the drifts is part of the preparation of the drift before start emplacement work;
- construction of the sealing plug but these activities is similar as for the KBS-3V design.

Table 2, presents a **"what-if"** analysis for this process.

5 Conclusions about Identified Failure Risks During Emplacement of the Super Container and Distance Blocks

Compared to the KBS-3V design the following advantage is identified:

- The lifting of the SC is done with water cushions and the lifting height is very low. No risk for dropping the waste package.
- No backfilling of large deposition drifts, about 6 000 m^3 each, is required.

The following disadvantages from a safety/availability point of view are identified:

- The consequences of malfunction of the deposition equipment including fire need to be investigated further and the small diameter of the drift makes it difficult to carry out correcting actions. However, the deposition machine should in all cases be possible to take back for repair if anything happens.
- If the ground water flow into the drift is too high, piping in the buffer may occur. Installations of compartment steel plugs in a drift may solve this problem but it is still a risk that must be investigated further. The requirements on ground water management are higher for KBS-3H then for KBS-3V.
- The events that can have severe impacts on availability are more in the KBS-3H design, since this is more technologically complex than 3V.

Table 2: What-if for deposit area

