


# A New Uncertainty Reduction Method for Fuel Fabrication Process and PWR cores with Erbia-Bearing Fuel



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# 1. Introduction

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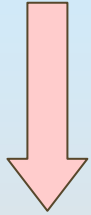
To achieve the super high burnup



- Increasing the  **$^{235}\text{U}$**  enrichment

The  **$^{235}\text{U}$**  enrichment must be less than 5wt% in Japan

## Erbia high burnup fuel



- **235U enrichment : >5wt%**
- **Erbia :  $\text{Er}_2\text{O}_3$**

## Fuel Fabrication System, Reactor Core

- The erbia addition to the fuel with over 5wt% 235U enrichment retains the neutronics characteristics to those with under 5wt% fuel.

- We have no experience for the reactor cores and the fuel fabrication systems with full loaded erbia
- The use of erbia may produce additional uncertainty of the neutronics characteristics

## ○ The Uncertainty of neutronics characteristics

### ■ The cross section error

➔ Evaluating with the **sensitivity coefficient** and the **cross section covariance matrix**

### ■ The calculational method error

#### ■ Monte-Carlo method

➔ The standard deviation of the neutronics characteristics

#### ■ Deterministic method

● The self-shielding model

➔ ● The energy condensation

● The calculational mesh divisions

### ■ The production error ( manufacturing tolerance )



## ○ The Uncertainty evaluation

$$R^c = R_t \left( 1 + \underbrace{G_R \Delta \sigma}_{\text{Cross section error}} + \underbrace{\Delta M}_{\text{Calculation method error}} + \underbrace{\Delta P}_{\text{Production error}} \right)$$

$\Delta \sigma$  : relative cross section error  $\left( = \frac{d\sigma}{\sigma} \right)$

### ■ The sensitivity coefficient

$$G_R = \frac{dR}{R} / \frac{d\sigma}{\sigma}$$

where  $R$  denotes the  $\rho$ ,  $k_{eff}$ , ...

The variance of  $R^c$

$$Var(R^c) = G_R \underbrace{V_x}_{\text{Cross section covariance matrix}} G_R^t + Var(\Delta M) + Var(\Delta P)$$

**Cross section covariance matrix**

# Improvement of prediction accuracy

## 1) Bias Factor Method

- Bias Factor =  $\frac{\text{Measured data of critical experiment}}{\text{Calculated data of critical experiment}}$
- Bias factor is multiplied to calculated neutronics characteristics for a target system to improvement the accuracy
- Simple and easy to apply to design calculations

## 2) Cross Section Adjustment Method

- The method can utilize a wide range of neutronics characteristics
- Use of many experimental data
- Reliable cross section

## 2. Computational Method

- The effective neutron multiplication factor of the erbia bearing fuel system

$$\frac{1}{k_{eff}} = \rho + \frac{1}{k_{eff,0}}$$

$\rho$  : the erbia worth

$k_{eff,0}$  : the neutron multiplication factor of the system without the erbia

- The uncertainty of the  $k_{eff}$

$$\frac{1}{(k_{eff})^2} d(k_{eff}) = -\rho G_{\rho} \Delta\sigma + \frac{1}{k_{eff,0}} G_{k_{eff,0}} \Delta\sigma$$





## ■ The variance of the $k_{eff}$

**The uncertainty of the erbia worth**

$$Var(k_{eff}) = k_{eff}^2 (G_1 V_{Er} G_1^t + G_2 V_{\sigma} G_2^t)$$

The uncertainty of the  $k_{eff,0}$



**Using the bias factor method or the generalized bias factor method**

$$G_1 = G_{\rho} \rho \quad G_2 = \frac{1}{k_{eff,0}} G_{k_{eff,0}}$$

$V_{Er}$  : The cross section covariance matrix of Er

$V_{\sigma}$  : The cross section covariance except for Er

## ■ Bias factor method

### □ The $k_{eff,0}$ of the target core

$$\tilde{k}_{eff,0} = k_{eff,0} \times f$$

Bias factor

The measured  $k_{eff,0}$   
for the mockup core

$$f = \frac{k_{eff,0,e}^e}{k_{eff,0,e}^c}$$

The calculated  $k_{eff,0}$   
for the mockup core



$$\tilde{k}_{eff,0} = k_{eff,0}^0 \left( 1 + G_{r,k_{eff,0}} \Delta\sigma \right) \times \frac{1 + \Delta E_k}{1 + G_{e,k_{eff,0}} \Delta\sigma}$$

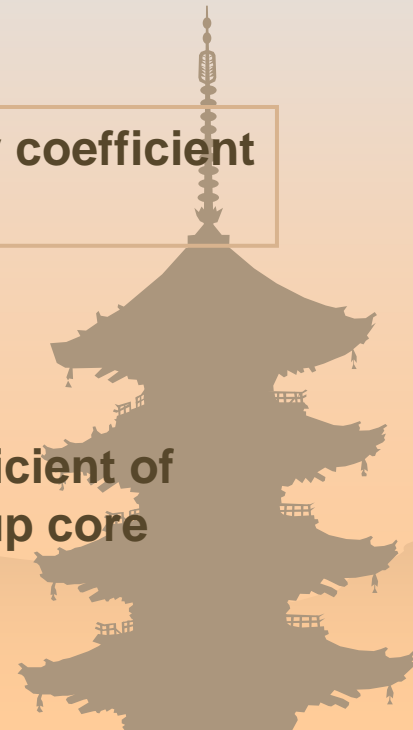
The sensitivity coefficient  
of the system

### □ The variance of the $\tilde{k}_{eff,0}$

$$Var(\tilde{k}_{eff,0}) = \Delta G_k V_\sigma \Delta G_k^t + Var(E_k)$$

$$\Delta G_k = G_{k_{eff,0}} - G_{e,k_{eff,0}} \quad G_{e,k_{eff,0}} : \text{the sensitivity coefficient of experimental mockup core}$$

$$Var(E_k) : \text{the variance of the measured } k_{eff,0,e}^e$$



## ■ Generalized bias factor method

### □ N experimental data

$$f_i = \frac{k_{eff,c,i}^e}{k_{eff,e,i}^e} \quad (i=1,2,3,\dots,N)$$

### □ **Generalized bias factor** is determined by

$$\tilde{f} = \sum_{i=1}^N C_i f_i$$

### □ **Weighting factor**

$$\sum_{i=1}^N C_i = 1.0$$



- The  $k_{eff,0}$  of the target core by generalized bias factor method

$$\tilde{k}_{eff,r} = k_{eff,r,0} (1 + G_k \Delta\sigma) \times \sum_{i=1}^N C_i \frac{1 + \Delta E_i}{1 + G_{k,i} \Delta\sigma}$$

- The variance of the  $\tilde{k}_{eff,0}$

$$\begin{aligned} Var(\tilde{k}_{eff,r}^c) &= \left( G_{k,r} - \sum_{i=1}^N C_i G_{k,i} \right) V_x \left( G_{k,r} - \sum_{i=1}^N C_i G_{k,i} \right)^t \\ &+ \sum_{i=1}^N \sum_{j=1}^N C_i C_j Var(\Delta E_i \cdot \Delta E_j) \end{aligned}$$



- The uncertainty of the erbia worth is reduced by using **the cross section adjustment method**

→ Utilizing the erbia sample worths measured by critical experiment

- **The adjusted cross section**

$$\sigma = \sigma_0 \left\{ 1 + V_{Er} G_{e,\rho}^t \left[ G_{e,\rho} V_{Er} G_{e,\rho}^t + Var(E_\rho) \right]^{-1} \frac{[\rho_e^e - \rho_e^c(\sigma_0)]}{\rho_e^c(\sigma_0)} \right\}$$

$\sigma_0$  : the cross section before adjustment

$Var(E_\rho)$  : the variance of the measured erbia sample worth

$G_{e,\rho}$  : the sensitivity coefficient of the erbia sample worth

$\rho_e^e$  : the measured erbia sample worth

$\rho_e^c(\sigma_0)$  : the erbia sample worth calculated by

- **The covariance of the adjusted cross section**

$$V_{Er}' = V_{Er} - V_{Er} G_{e,\rho}^t \left[ G_{e,\rho} V_{Er} G_{e,\rho}^t + Var(E_\rho) \right]^{-1} G_{e,\rho} V_{Er}$$

- The variance of  $\rho_r^c(\sigma)$

$$\text{Var}[\rho_r^c(\sigma)] = G_{r,\rho} V_{Er} G_{r,\rho}^t$$

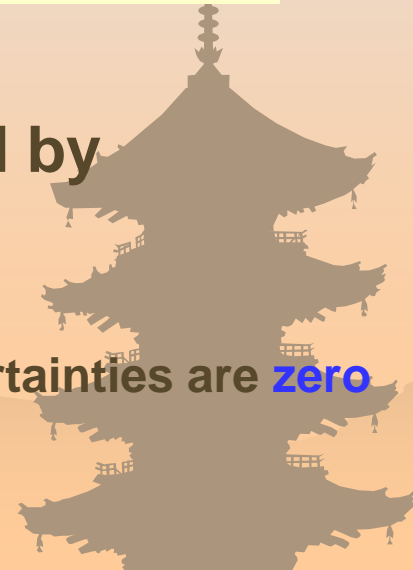
- The variance of the effective neutron multiplication factor in the target system

$$\text{Var}(\tilde{k}_{eff}) = k_{eff}^2 \left\{ G_{r,\rho} V_{Er} G_{r,\rho}^t \rho^2 + \frac{1}{k_{eff,0}^2} [\Delta G_k V_\sigma \Delta G_k^t + \text{Var}(E_k)] \right\}$$

- The uncertainty reduction (UR) is defined by

$$UR = 1 - \frac{\text{Var}(\tilde{k}_{eff})}{\text{Var}(k_{eff})}$$

When UR becomes **1.0**, the uncertainties are **zero**



# 3. Computational Model

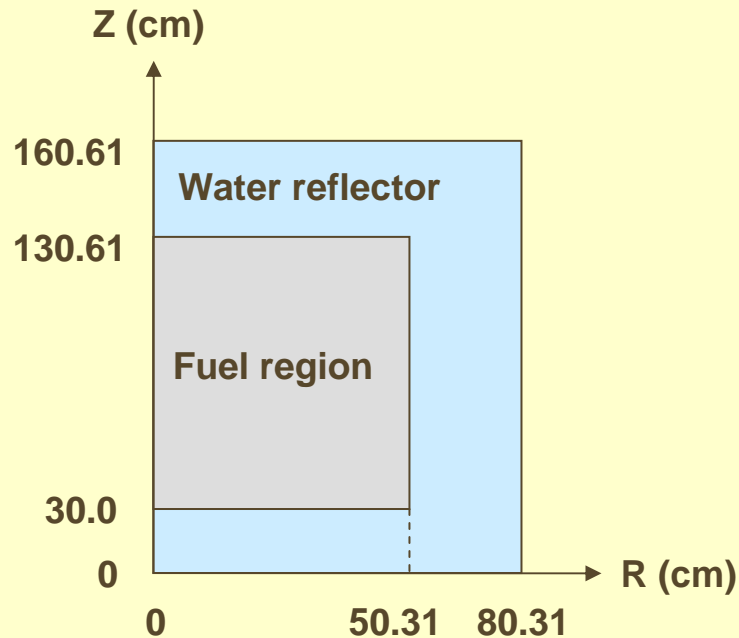


Fig. Computational geometry of a blending machine (target system)

- $^{235}\text{U}$  enrichment : **5.4wt%**
- The enrichment of Er relative to U : **0.3wt%**
- Temperature : **300K**
- H/U = **0 , 1**

- Cross section set : **JENDL-3.3**
- Sensitivity coefficient : **SAGEP**

***Blending machine***

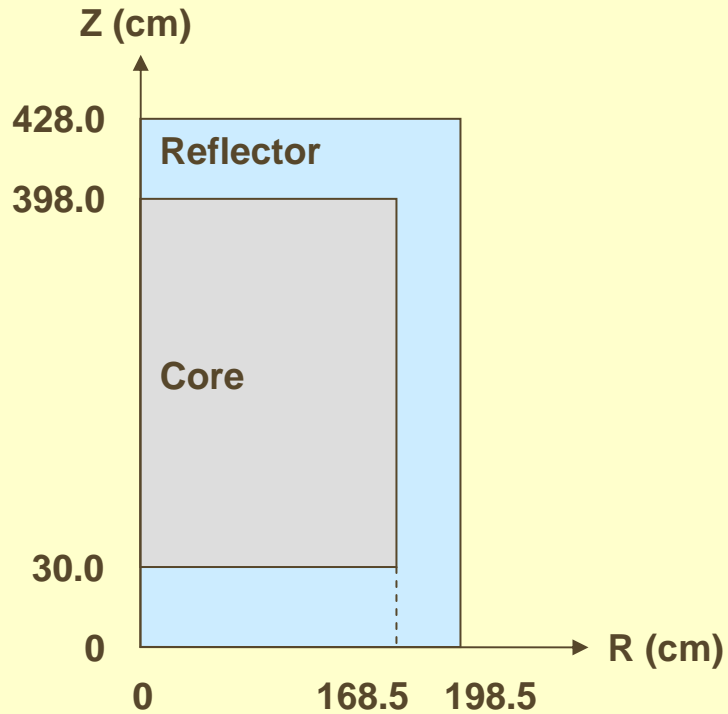


Fig. Calculational geometry of a PWR core (target system)

- $^{235}\text{U}$  enrichment : **5.4wt%**
- The enrichment of Er relative to U : **0.3wt%**
- Temperature : **300K**

- Cross section set : JENDL-3.3
- Sensitivity coefficient : SAGEP

**PWR core**







Fig. Erbia plate

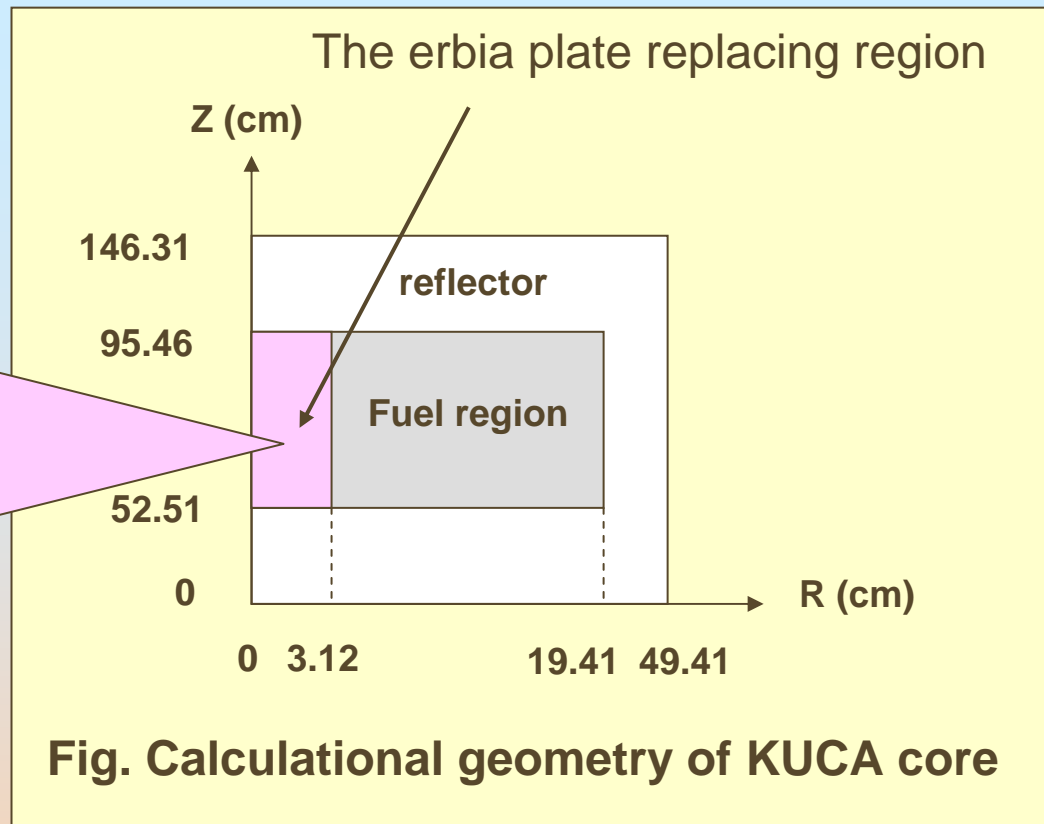


Table Calculated erbia sample worth and C/E value of KUCA experiment

Number of erbia plates	Calculated sample Worth ( $\Delta k/k$ )	C/E
3	$4.11 \times 10^{-4}$	1.09
5	$7.07 \times 10^{-4}$	1.13
9	$1.27 \times 10^{-3}$	1.11
13	$1.44 \times 10^{-3}$	1.12
17	$1.67 \times 10^{-3}$	1.10

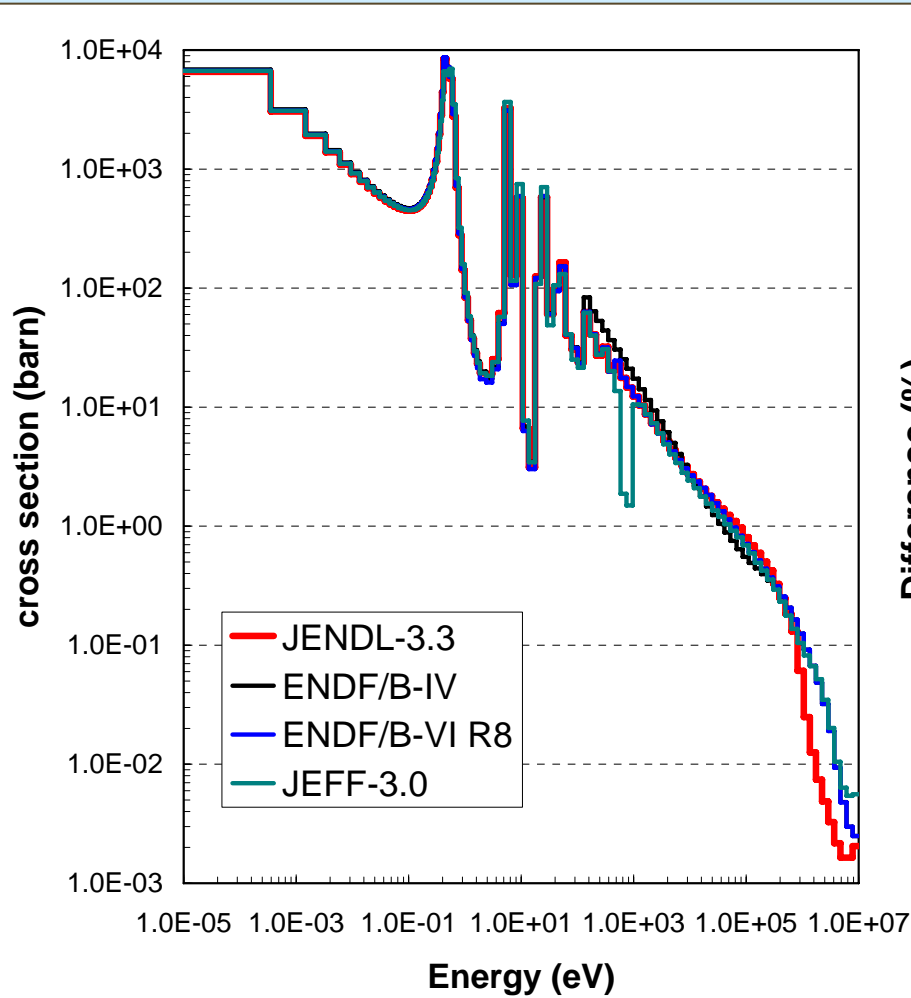


Fig. Comparison of  $^{167}\text{Er}$  capture cross section in each nuclear data library

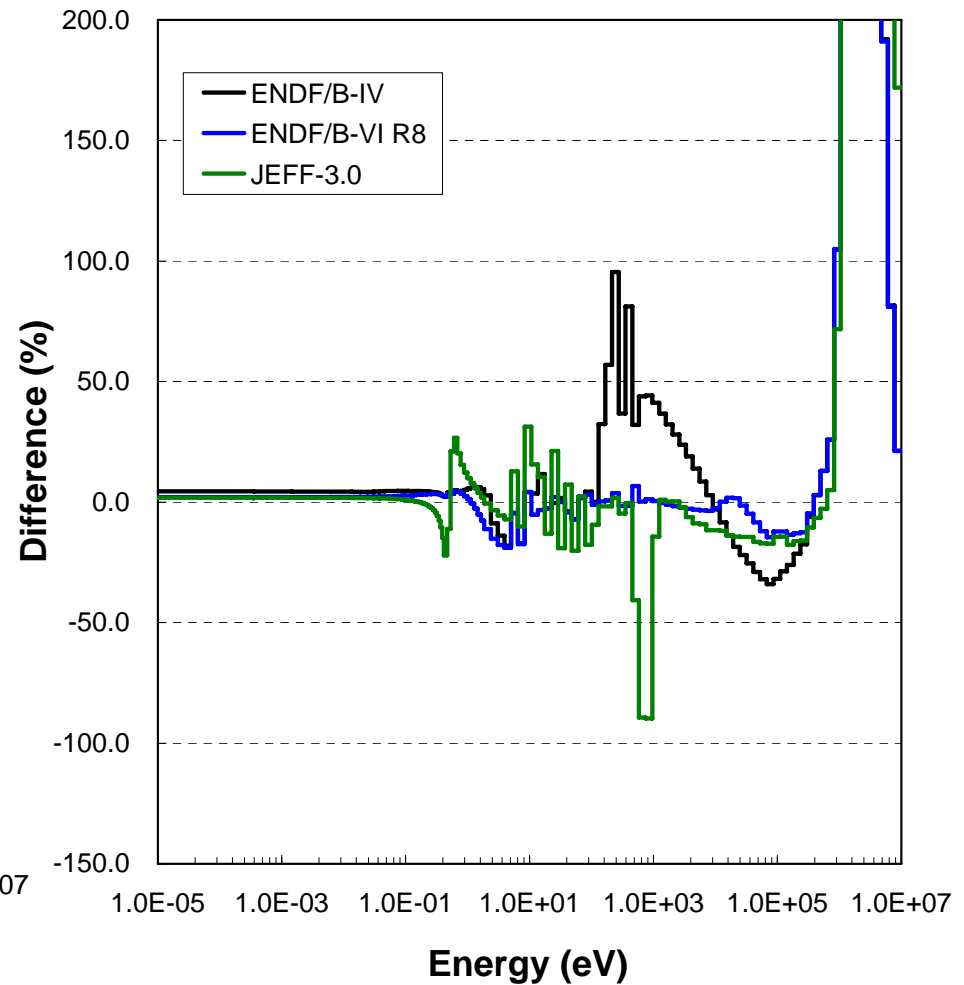


Fig. Difference of  $^{167}\text{Er}$  capture cross section from JENDL-3.3

**Er cross section errors → 10%**

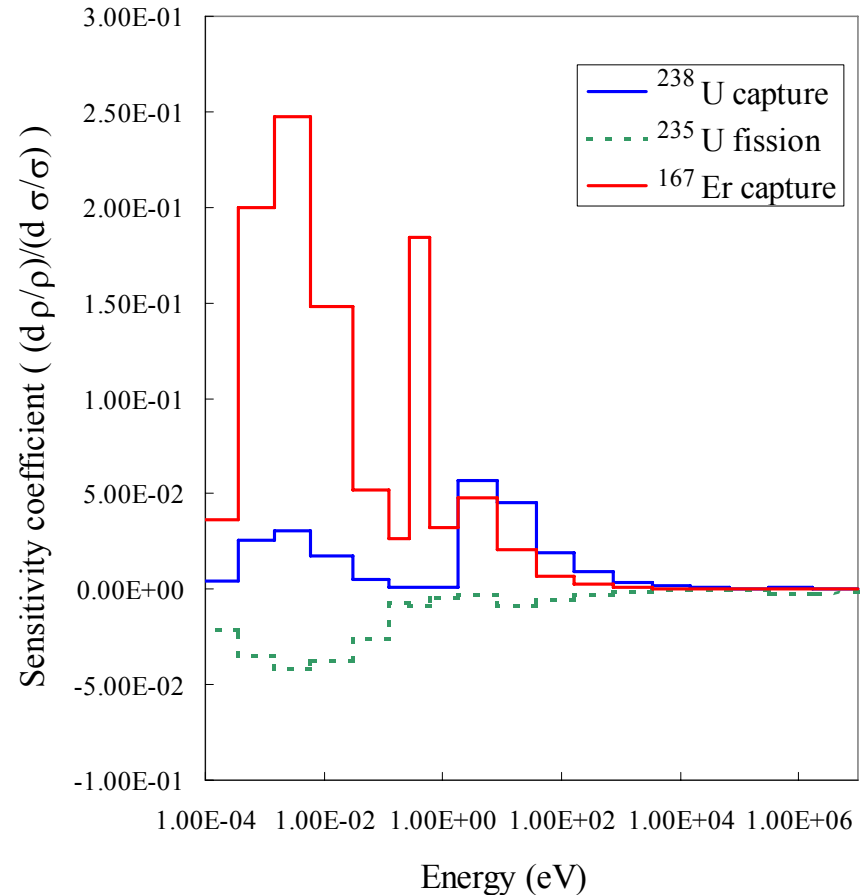
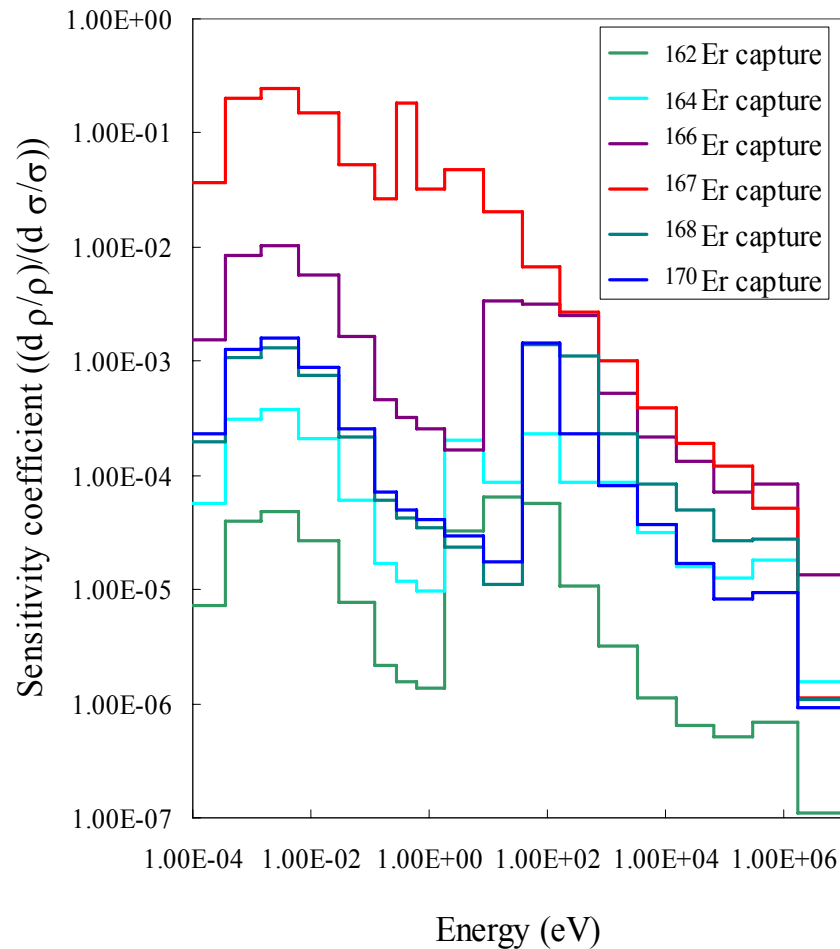
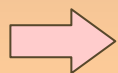
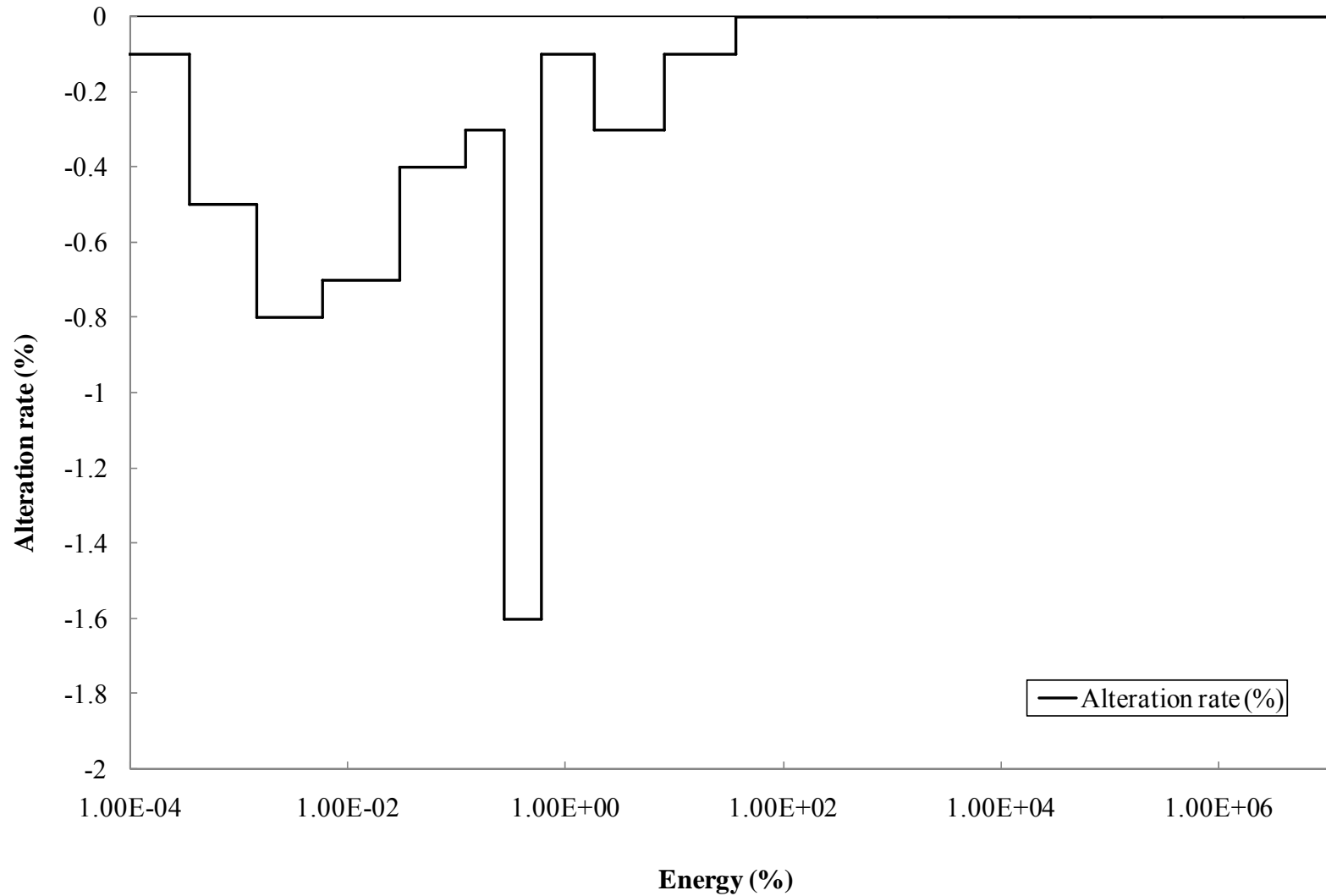


Fig. Comparison of sensitivity coefficients for erbia reactivity worth in KUCA



**167Er capture cross section is adjusted**



**Fig. Alteration rate of  $^{167}\text{Er}$  capture cross section due to adjustment**

# 4. Calculation Results

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- Evaluation of uncertainty reductions

**Blending machine** in fuel fabrication process

**PWR core**

- Method of uncertainty reduction

**Present method**

Erbia worth : cross section adjustment method

keff : Generalized bias factor method

➡ Two critical experiment data **without erbia**

**Bias factor method**

➡ critical experiment data **with erbia** is directly applied

**Without Experiment Data**



## Case of a blending machine

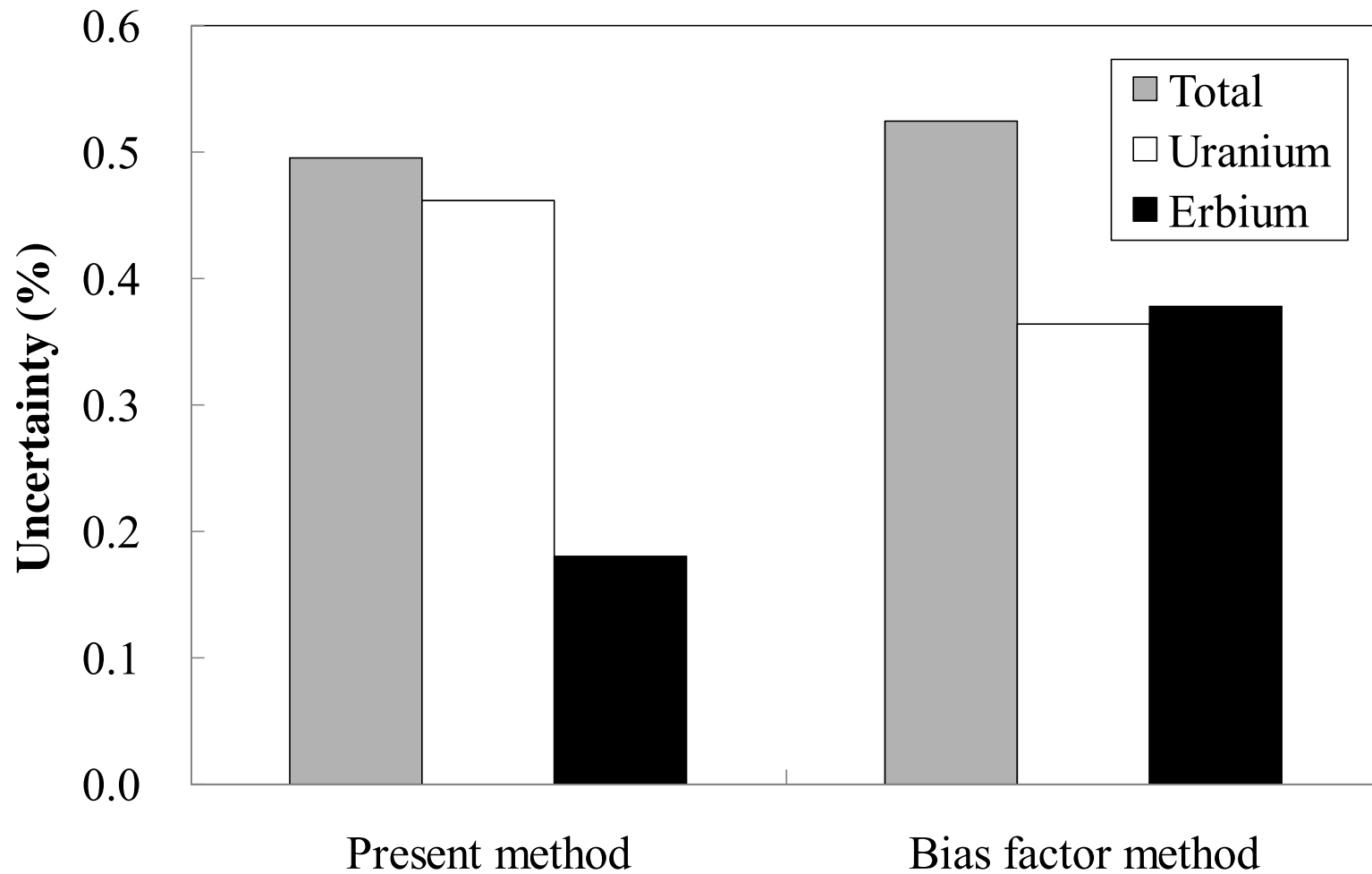
Table Comparison of prediction uncertainties and uncertainty reduction (UR) for  $k_{eff}$  in a blending machine ( $H/U=0$ )

	Uncertainty (%)	UR
<b>Present method</b>	<b>0.495</b>	<b>0.604</b>
Bias factor method	0.525	0.555
Case without experimental data	0.787	0.000

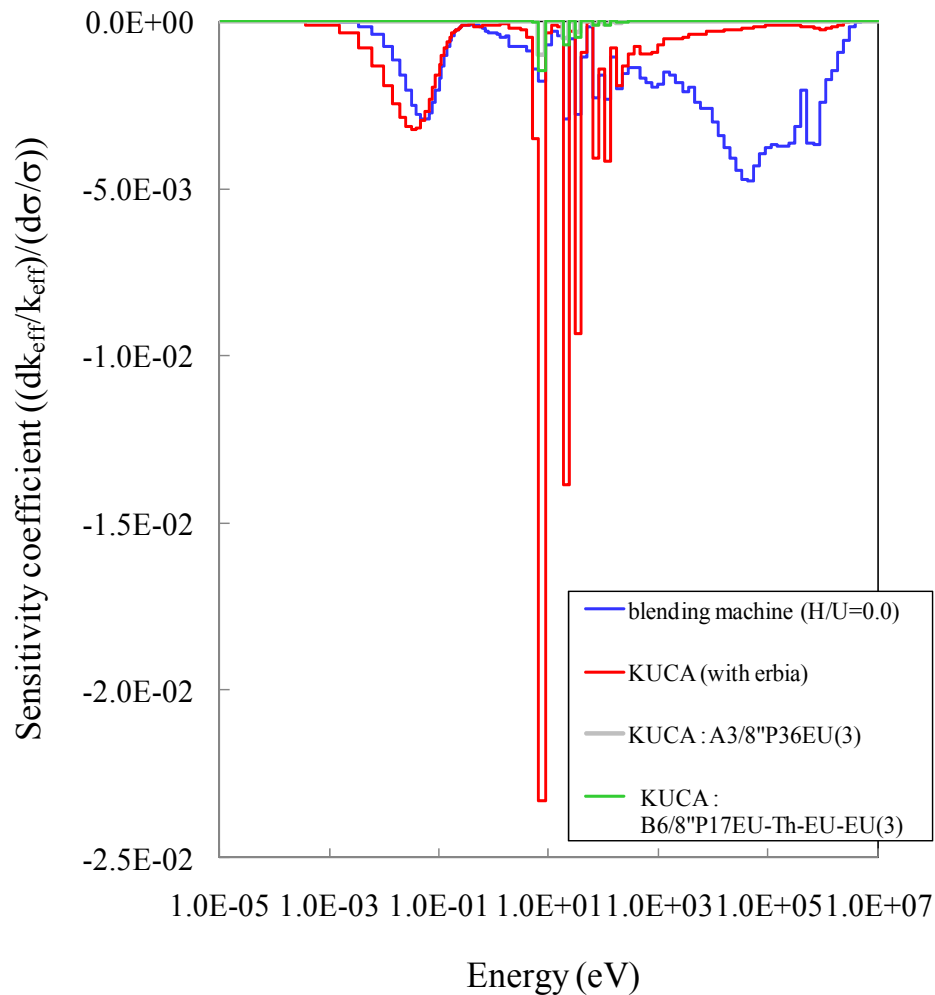
$$k_{eff} = 0.6210$$

$$k_{eff,0} = 0.6361$$

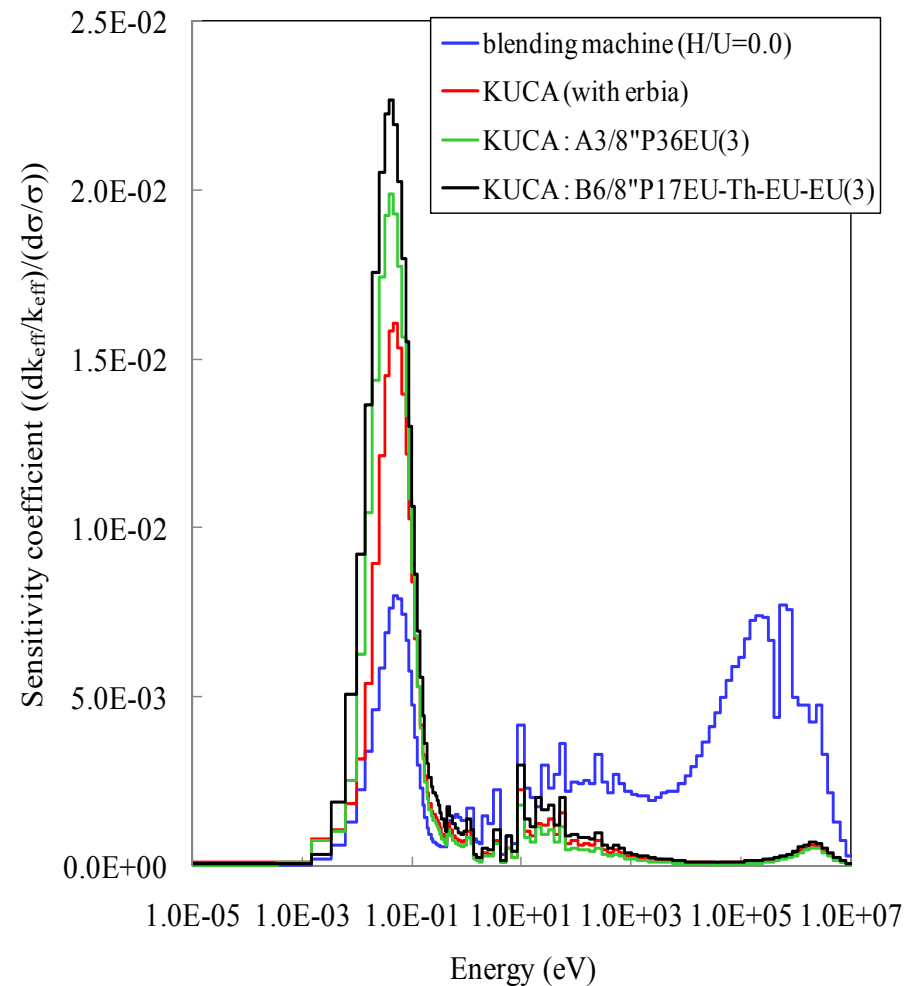
$$\rho = 3.823 \times 10^{-2} (\Delta k/k)$$



**Fig. Components of prediction uncertainty of keff in a blending machine (H/U=0.0)**



**Fig. Sensitivity coefficients of  $k_{eff}$  with respect to  $^{238}\text{U}$  capture cross section**



**Fig. Sensitivity coefficients of  $k_{eff}$  with respect to  $^{235}\text{U}$  fission cross section**



**Table Comparison of prediction uncertainties and uncertainty reduction (UR) for  $k_{eff}$  in a blending machine (H/U=1)**

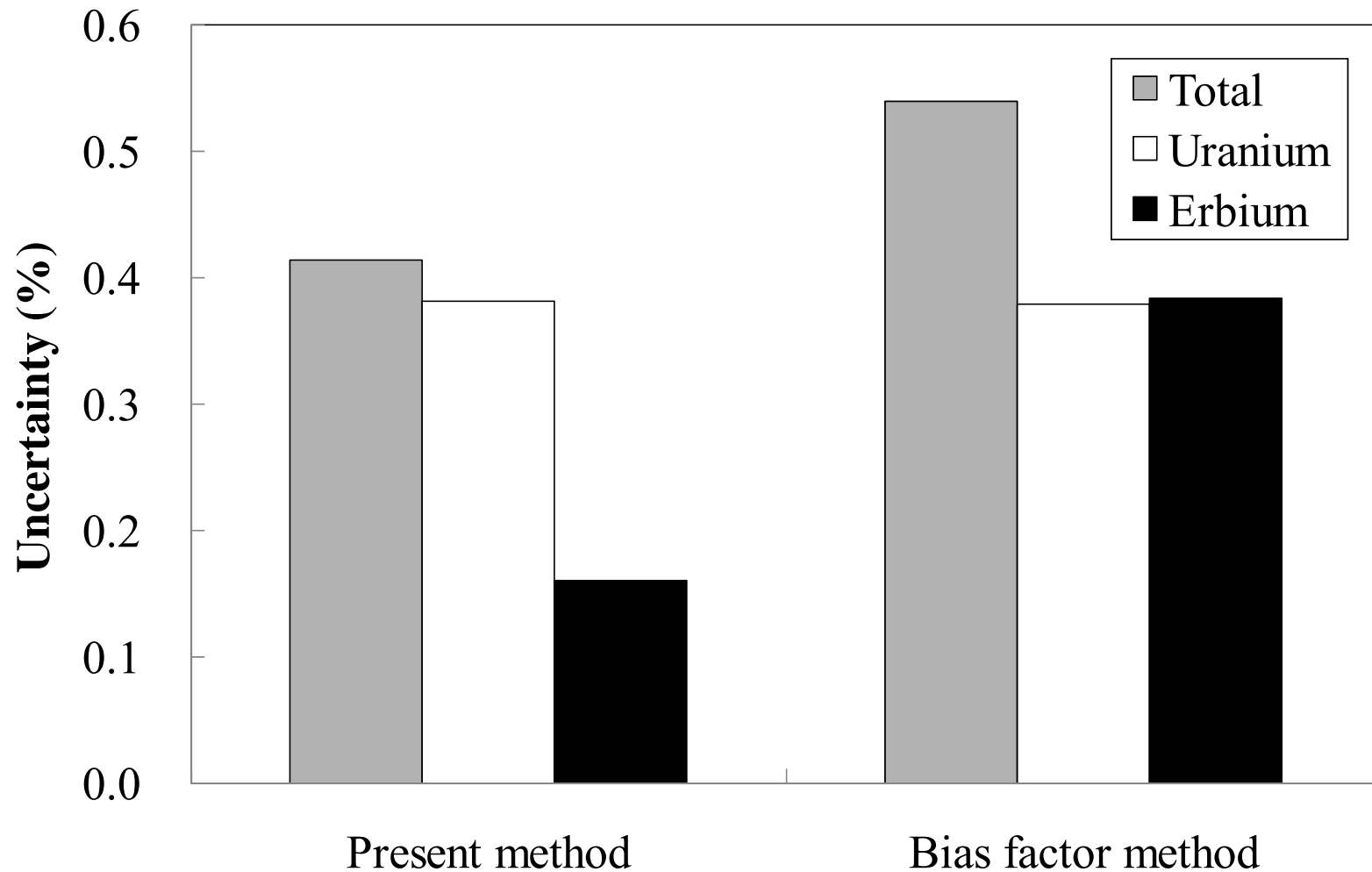
	Uncertainty (%)	UR
<b>Present method</b>	<b>0.414</b>	<b>0.760</b>
Bias factor method	0.539	0.593
Case without experimental data	0.845	0.000

$$k_{eff} = 0.8026$$

$$k_{eff,0} = 0.8287$$

$$\rho = 3.924 \times 10^{-2} (\Delta k/k)$$





**Fig. Components of prediction uncertainty of keff in a blending machine (H/U=1)**

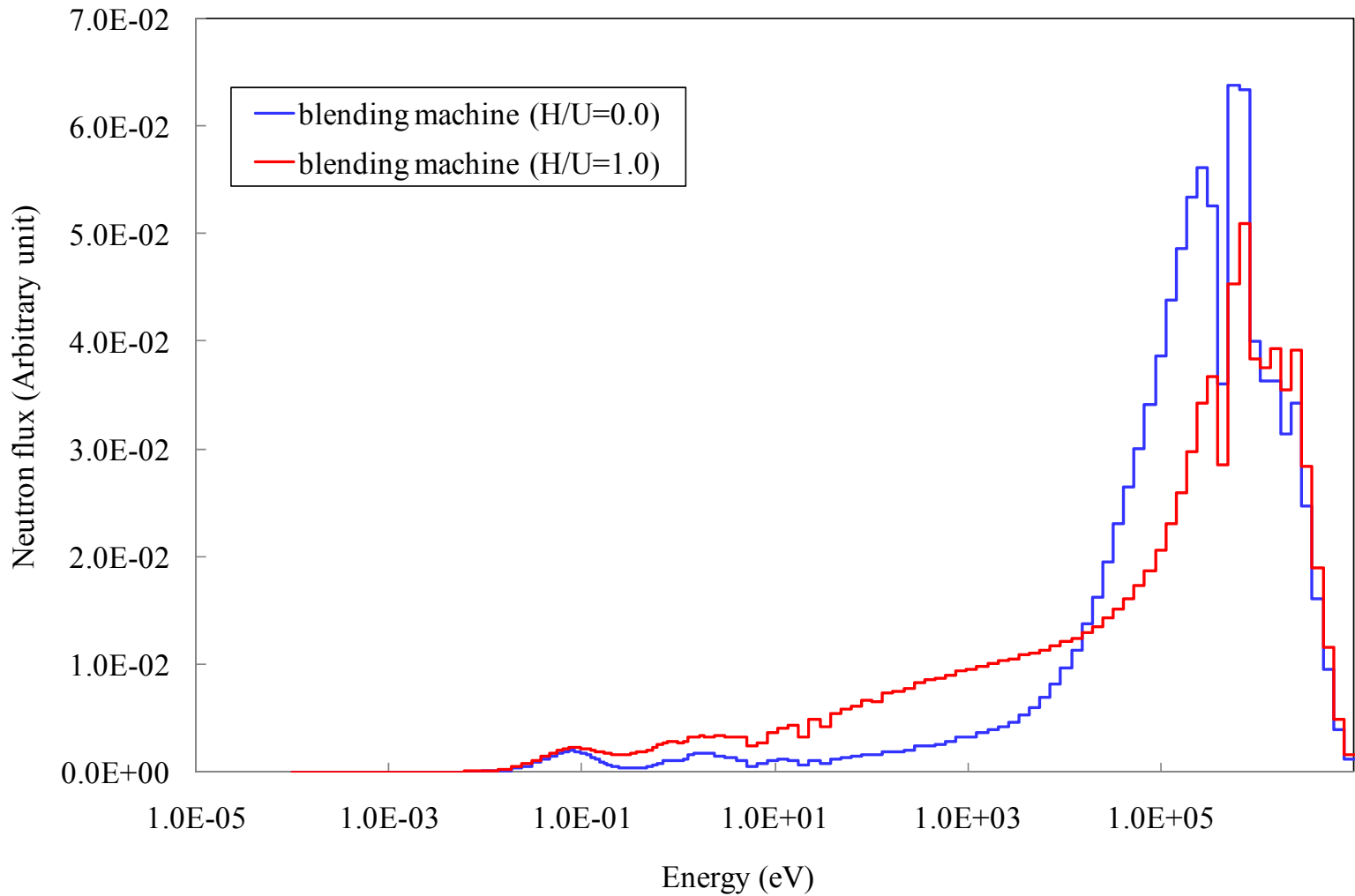


Fig. Comparison of neutron spectrum  
between the case of H/U=0.0 and H/U=1.0

## Case of a PWR core

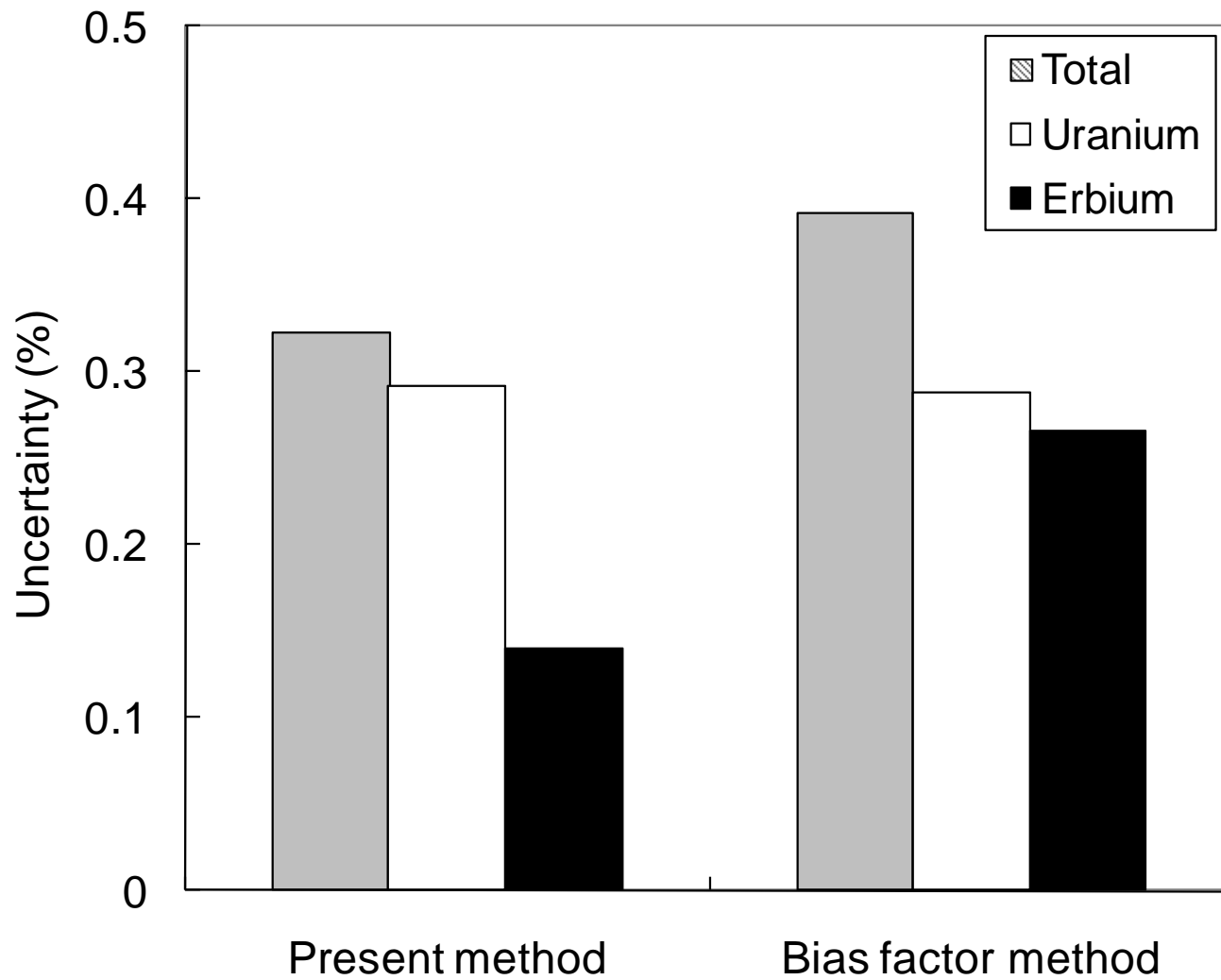
Table Comparison of prediction uncertainties and uncertainty reduction (UR) for  $k_{eff}$  in a PWR core

	Uncertainty (%)	UR
<b>Present method</b>	<b>0.322</b>	<b>0.865</b>
Bias factor method	0.392	0.801
Case without experimental data	0.879	0.000

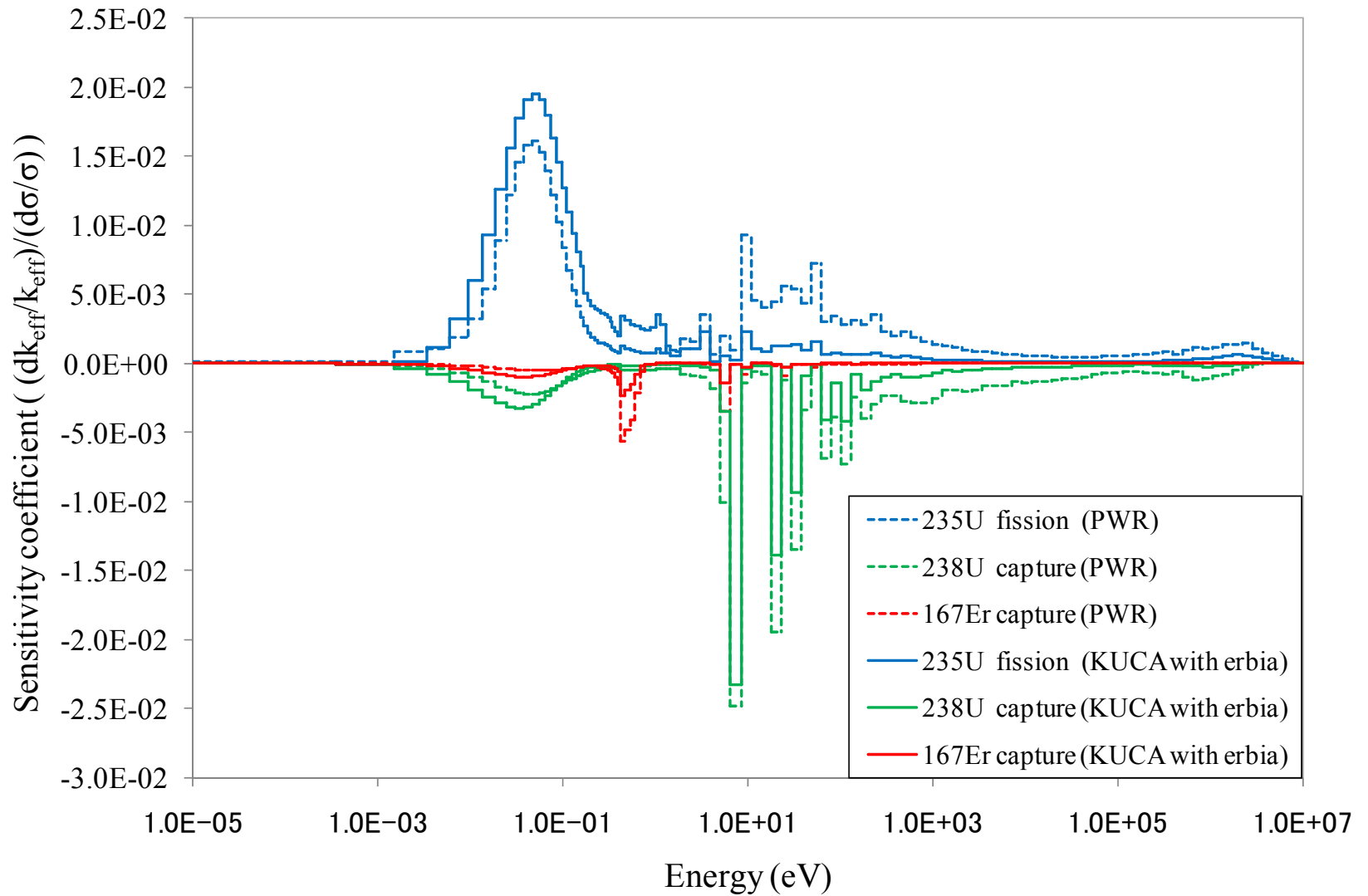
$$k_{eff} = 1.0072$$

$$k_{eff,0} = 1.0479$$

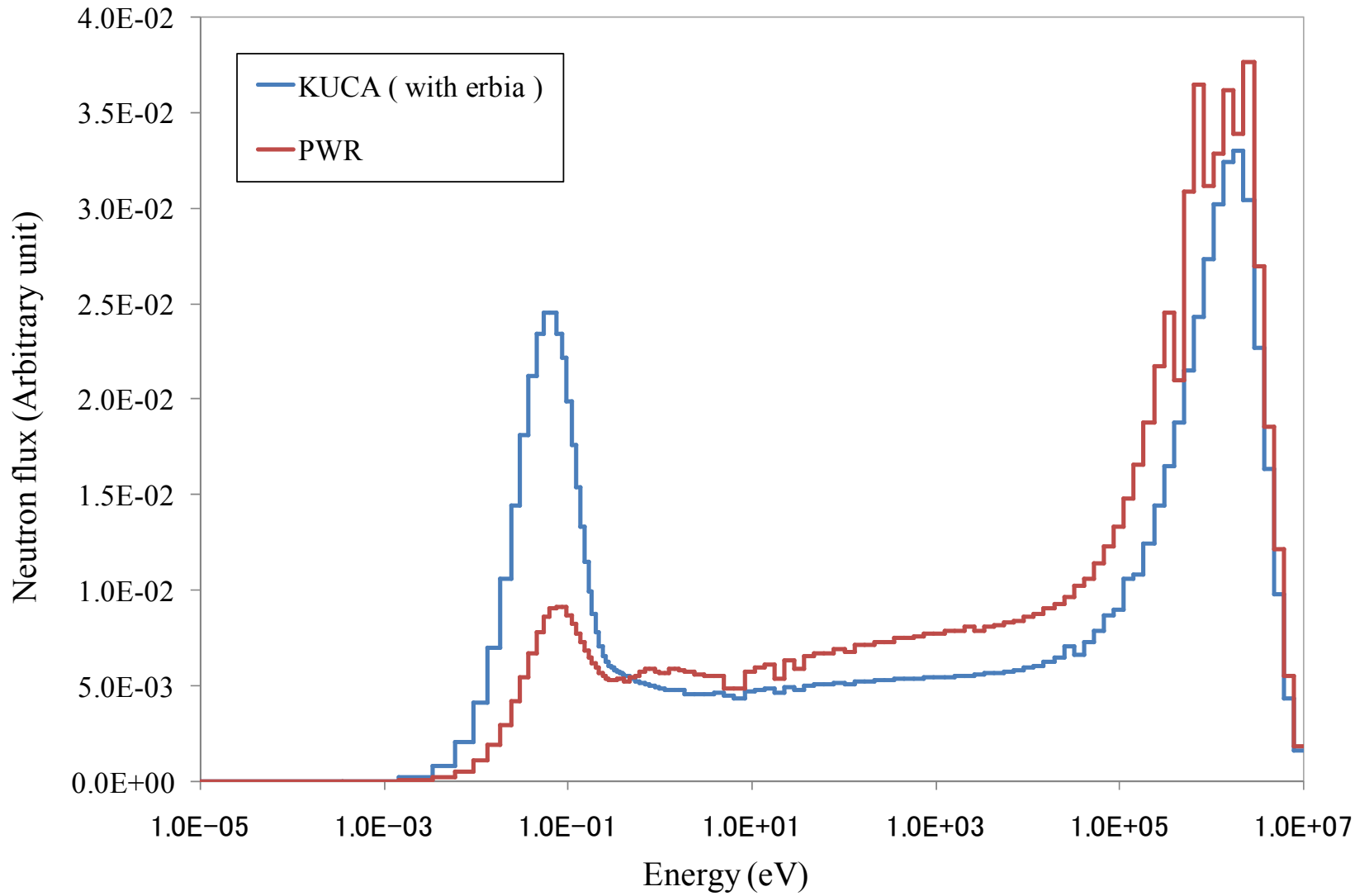
$$\rho = 3.856 \times 10^{-2} (\Delta k/k)$$



**Fig. Components of prediction uncertainty of keff in a PWR core**



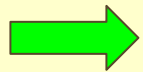
**Fig. Sensitivity coefficients of keff for a PWR core and KUCA**



**Fig. Comparison of neutron spectrum**

## 5. Conclusion

- A new method was proposed.



**Combining the bias factor method and the cross section adjustment method**

- The present method was applied to evaluate the prediction uncertainty of neutronics characteristics of **the blending machine and the PWR core loading the erbia bearing fuel.**

- The uncertainty of erbia worth was reduced through **the cross section adjustment method using the erbia sample worth measured in KUCA.**



## ***Blending machine***

### ■ **H/U=0**

UR = **0.604** by **the present method**

UR = 0.555 by the bias factor method

### ■ **H/U=1**

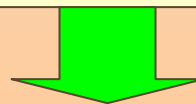
UR = **0.760** by **the present method**

UR = 0.593 by the bias factor method

## ***PWR core***

UR = **0.865** by **the present method**

UR = 0.801 by the bias factor method



**The uncertainties were reduced by the present method**

***Thank you for your attention!!***

***Kyoto University Research Reactor Institute***

