# **Homogeneity study of matrix constituents of a fuel plate through gamma ray transmission**

**By**

**Muhammad Usman Rajput Muhammad Akhtar Javaid Iqbal Akhtar**

*Physics Division Directorate of Science Pakistan Institute of Nuclear Science and Technology Nilore, Islamabad*

**December, 2008**

## *Table of Contents*



 $\sim$ 

 $\sim 10$ 

## **Abstract**

The R & D work to establish a facility for the study of homogeneity and fuel density i.e., quantity of  $^{235}U$  in the locally fabricated LEU fuel plates for Pakistan Atomic Research Reactor (PARR-1) has been initiated. This brief report presents the results of our preliminary study in respect of homogeneity of the fuel plate constituents. An elementary experimental facility for nuclear radiation detection with Nal detector has been established. The fuel plate  $(U_3Si_2-AI)$  was manually scanned and the transmission of gamma radiation from radioactive <sup>241</sup>Am source was determined. The study revealed encouraging results about density distribution of the plate constituents. At the same time it has also lead to future improving steps.

Key words: nuclear fuel, gamma ray interaction, gamma ray scanner

 $\frac{1}{2}$ 

#### **1. Introduction**

Ĵ

Nuclear fuel plays a key role in smooth and successful running of the nuclear reactors. The homogenous distribution of the fissile material in the fuel is an important parameter in the fuel fabrication technology. The inhomogeneous distribution may lead to the formation of high temperature regions in the fuel plates and elements. These regions are called hotspots and may lead to the catastrophic failure of the reactor. The qualification of nuclear fuel therefore becomes very important.

Gamma ray spectroscopic technique serves as a powerful and reliable tool in the qualification of nuclear fuel. The gamma rays of interest to non-destructive analysis fall within the range of 10 keV to 2000 keV depending on the type and geometry of the materials to be scanned. The gamma rays interact with the detectors and the absorbing media (i.e., the fuel plate) by three major processes [1-4], namely, the photoelectric effect, Compton scattering and pair production. In the photoelectric effect, the gamma ray loses all of its energy in the single collision with the bound electrons in the absorbing material. The probability of interaction depends very strongly on the gamma ray energy and the atomic number. The Compton scattering results in the partial loss of energy and the gamma ray suffers multiple collisions. The probability in this process is weakly dependent on the energy and the atomic number. In the pair production process, the threshold for the gamma ray energy is 1.02 MeV which is the rest mass of the two generated electron-positron pair which is equal to  $2 \times m_0C^2$ .

All the above mentioned processes contribute to the complex gamma ray spectrum, especially for the multi gamma emitting nuclides. Based on gamma ray interaction and attenuation phenomenon, gamma ray scanners can be designed. The design and development work of the gamma ray scanner for homogeneity measurement is progressing successfully. This report presents our R & D detail and the results of the scan of one of the fuel plates.

l

### **2. Experimental Set-up of LEU Fuel Gamma Ray Scanning System**

In order to develop a gamma ray scanning system, the following facilities were established at PINSTECH.

- **Establishment of gamma ray detection/scanning system for fuel plate** *homogeneity measurements*
- *Designing and fabrication of tentative partial lead shielding*
- *Signal processing*
- *Manual Scanning of fuel plate*
- *Marking of scan grids on the fuel plate*
- *Provision of 241 Am radioactive source (IAEA-standard)*
- *Fabrication of holder for radioactive source*

The layout of the designed gamma ray scanner is shown in Fig. 1 (a). The detection system comprising of Nal(TI) detector, a photomultiplier tube and pre-amp base along with signal cables is inside the lead brick surroundings. The fuel plate is shown in the scanning position in between the detection system and the <sup>241</sup>Am radionuclide source. The radionuclide is also placed inside the lead shielding in order to avoid the radiation hazard to the personnel during manually scanning the fuel plate. The allied electronic system coupled with the gamma ray detection system is shown in Fig. 1 (b). The block diagram of the detection system is shown in Fig. 2.



 $(a)$ 







Fig.2 : Block diagram of the gamma ray Scanning system

In order to mark the position for measurements and to keep the reproducible geometry conditions, a graphic sheet was attached on one side of the fuel plate. It was numbered (named as segments) vertically and horizontally. The following lines define the segments and measuring positions. The read convention is explained as under: -

Position (2, 12) defines segment #2 which is 2 cm vertical plate position and 12 cm horizontal position from the referenced corner of the fuel plate. In the graphical representation of the data in this report "comma" of position as 2, 12 has been omitted. This position will be taken as 212 for plotting convenience.



**Segment #6: (6,12), (6,14), (6,16), (6,18), (6,20), (6,22), (6, 24), (6, 26), (6, 28), (6, 30), (6, 32), (6, 34)**

**Segment #8: (8,12), (8,14), (8,16), (8,18), (8,20), (8,22), (8, 24), (8, 26), (8, 28), (8, 30), (8, 32), (8, 34)**

**The fuel plate grid is shown in Fig.3 (a & b). In the Fig.3 (a), face-1 shows the actual shape of the fuel plate where as face-2 in Fig.3 (b) indicates the pasted graphic sheet used for determining the scanned position and for reproducible remeasurements.**

The radioactive source of <sup>241</sup> Am (IAEA point source standard) was mounted on a **fabricated source holder (Fig. 4) and fixed in an appropriate position inside the lead shielding of the scanning system. Several wooden blocks were prepared to keep the fuel plate in stable condition. The plate was manually moved vertically and horizontally on the wooden blocks; however, an additional stable condition was provided by hand holding at certain positions.**



**Fig.3 : (a) Facel- Actual layout of the uranium silicide fuel plate (b) Face 2 gridding of the uranium silicide fuel plate.**



**Fig 4. IAEA radioactive standard source (Am-241) and the locally fabricated source holder.**

### **3. Gamma Ray Scanning of fuel plate**

**As the plate contained radioactive material, though of small activity, all the radiation safety measures were undertaken, during manual scanning for data collection. The detection system was optimized by incorporating the Oscilloscope in the electronic circuitry and a gamma ray window was set at single channel analyzer. Fuel plate was manually moved longitudinally and vertically and the transmission of 59 keV gamma ray of Am-241 was recorded at the grids marked earlier, by using the counter/timer of the scanning system. The random nature of** **radioactivity required several measurements at each position. The following procedure was adopted.**

- *Six data points were recorded at each position by manual scanning of the fuei plate (compiled in col. 2 to 6 in Appendix* - *I to IV). About 300 data sets were collected.*
- *Corresponding backgrounds were accumulated*
- *Data points were averaged for each position with an estimate of the corresponding uncertainty*.
- *The results of the data reduction are compiled. The results are also displayed graphically.*

### **4. Results and Discussion**

**The fuel plate was scanned at each segment i.e., 2, 4, 6 and 8 as defined earlier.** The data obtained at each segment is given as Appendix - I, II, III and IV **respectively. The results displayed in figures 6 and 7 are highly encouraging as they show the homogeneity of the fuel plate constituents at the corresponding positions. However, the figures 5 and 8 clearly reveal the existence large variations. The higher gamma transmission is noted in Figs. 5 and 8, whereas Figs. 6 and 7 shows a constant trend with reduced transmission indicating the presence of high-density constituents in the center of the fuel plate with homogeneous distribution. The possible causes of large variations noted from Figs. 5 and 8 were scrutinized and the possible reasons were: gamma ray streaming through the plate edges, the poor collimation and systematic errors that occurred as a result of manual scan. However, these errors were minimized in positions 6 and 7 owing to the central geometry of the plate that has masked the complete detector.**

**This preliminary study indicated the causes of discrepancies and that would lead to the following future improvements.**

- **1. Improvement in the collimation system.**
- **2. Requirement of 241 Am radioactive source of high activity.**
- **3. Electrically operated system for fine and stable horizontal/vertical movements of the fuel plates coupled with PC based data acquisition.**

**This work has been carried out using the only one available fuel plate, whereas better assessment requires few more fuel plates. Hence, further fuel plates with homogeneous and heterogeneous matrix material distribution will be required to conduct a comparative study in order to increase the confidence level. Uranium standards with known concentrations of 235U are also required to see the possibility of 235U quantification in the fuel plates.**

#### **5. References**

**[1], G.F. Knoll, Radiation Detection and Measurements- (3rd-Ed.) (John Wiley & Sons, New York, 1999)**

**[2]. K. Debertin and R.G. Helmer, Gamma- and X-Ray Spectrometry with Semiconductor Detectors. (North-Holland, Amsterdam, 1988)**

**[3]. S. Glasstone, A.Sesonske, Nuclear Reactor Engineering, (Van Nostrand Reinhold, New York, 1967)**

**[4]. G. Gilmore, J. Hemingway, Practical Gamma ray Spectrometry, (John Wiley & Sons Ltd, West Sussex, England, 2004)**

# **APPENDICES**

 $\frac{1}{2}$ 

 $\mathcal{L}^{\text{max}}_{\text{max}}$  and  $\mathcal{L}^{\text{max}}_{\text{max}}$ 

 $\label{eq:2.1} \frac{1}{\sqrt{2}}\left(\frac{1}{\sqrt{2}}\right)^{2} \left(\frac{1}{\sqrt{2}}\right)^{2} \left(\$ 

## **Appendix-1**



 $\mathcal{L}$ 







**Fig. 5: Preliminary results of fuel plate scan position 2**

## **Appendix-ll**





### Data reduction and Fig 2: <sup>241</sup> Am Source and Fuel plate segment # 4 scan



Fig. 6: Preliminary results of fuel plate scan position 4

Appendix-UI

## Measured counting data: 241/\m *Source and Fue1 plate segment* **#** 6 *scan*

 $\Lambda$ 



 $\sim$   $\sim$ 



### **Data reduction and Fig 3: <sup>241</sup> Am Source and Fuel plate segment #6 scan**



Fig. 7: Preliminary results of fuel plate scan position 6

**Measured counting data: <sup>241</sup>Am Source and Fuel plate segment # 8 scan** 

Appendix-IV



J.



## Data reduction and Fig 4: <sup>241</sup>Am Source and Fuel plate segment # 8 scan



Fig. 8: Preliminary results of fuel plate scan position 8